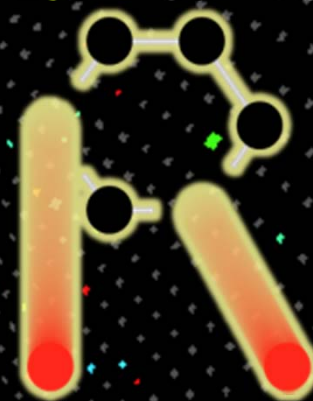


Taku Ishida
Chris J.Densham
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Shin Meigo



RADIATE
Collaboration

J-PARC Neutrino Beam-line Status & Radiation Damage Studies on Titanium Alloys

Current and Future J-PARC-Based Long-Baseline Neutrino Oscillation Experiments



T2K

Tokai-to-Kamioka

Kamioka Mine
Nucleon Decay Exp.
Neutrino Detection Exp.
 39mφ x 41mH
 Total[Fiducial]
 Volume
 = 50[22.5]kt



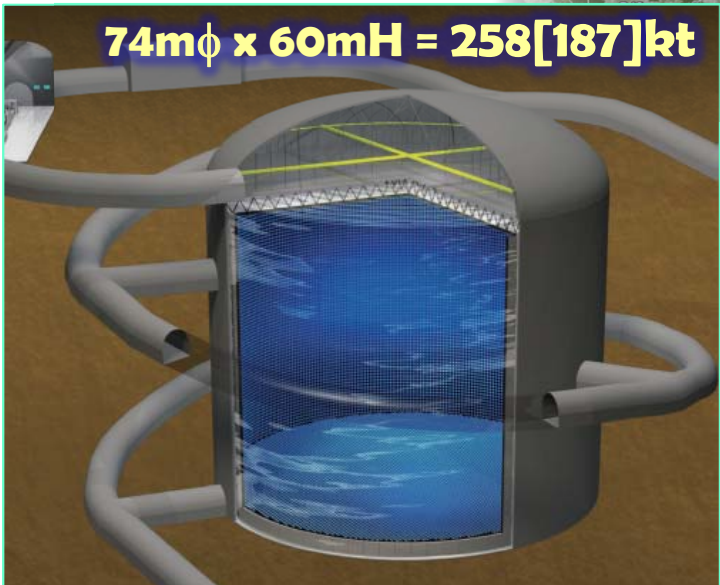
$\pi^\pm \rightarrow \mu + \nu_\mu$ 485kW achieved

upgrade to
1.3 MW

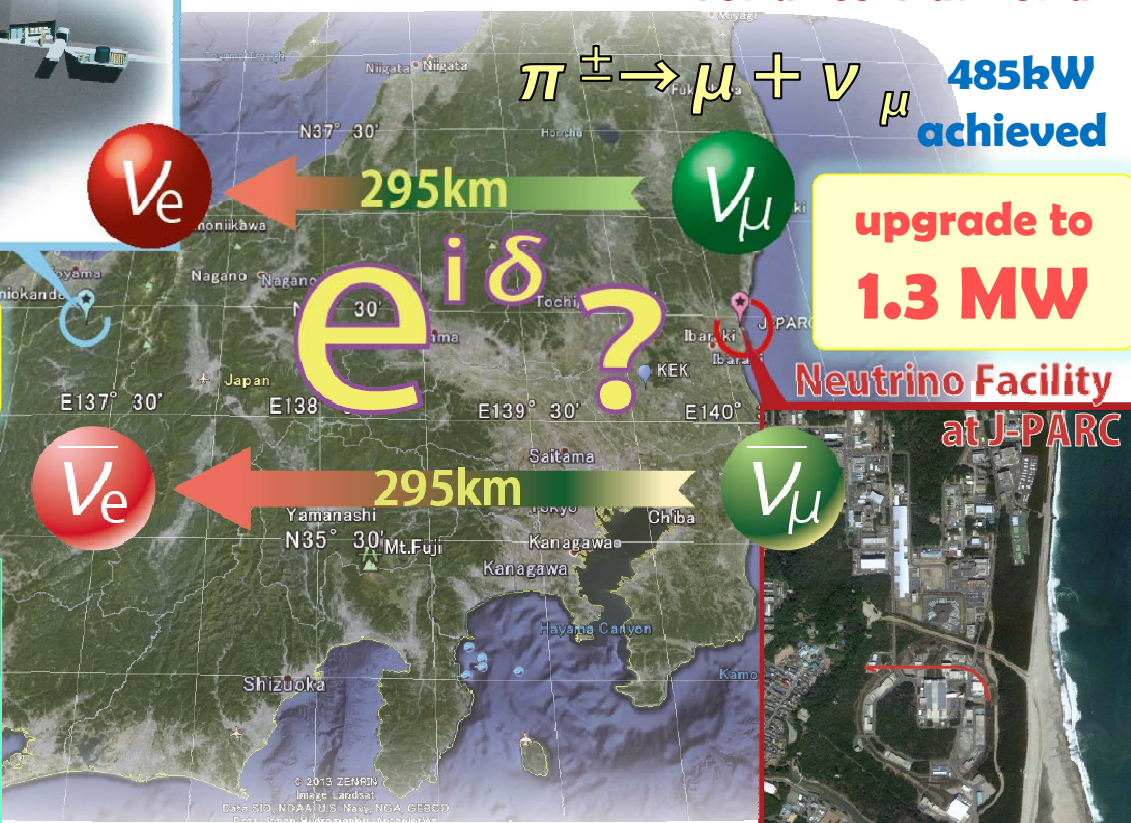
x ~10 of Super-K



Hyper-Kamiokande



74mφ x 60mH = 258[187]kt



Neutrino Facility
 at J-PARC

Tokai village

**Discovery of CPV in
 Lepton Sector**

400 MeV
H- Linac

3GeV Rapid Cycling
Synchrotron (RCS)
25Hz, 1MW

Neutrino
Experimental
Facility (ν)

Materials & Life
Science Facility
(MLF, MUSE)

A round: 1,568m

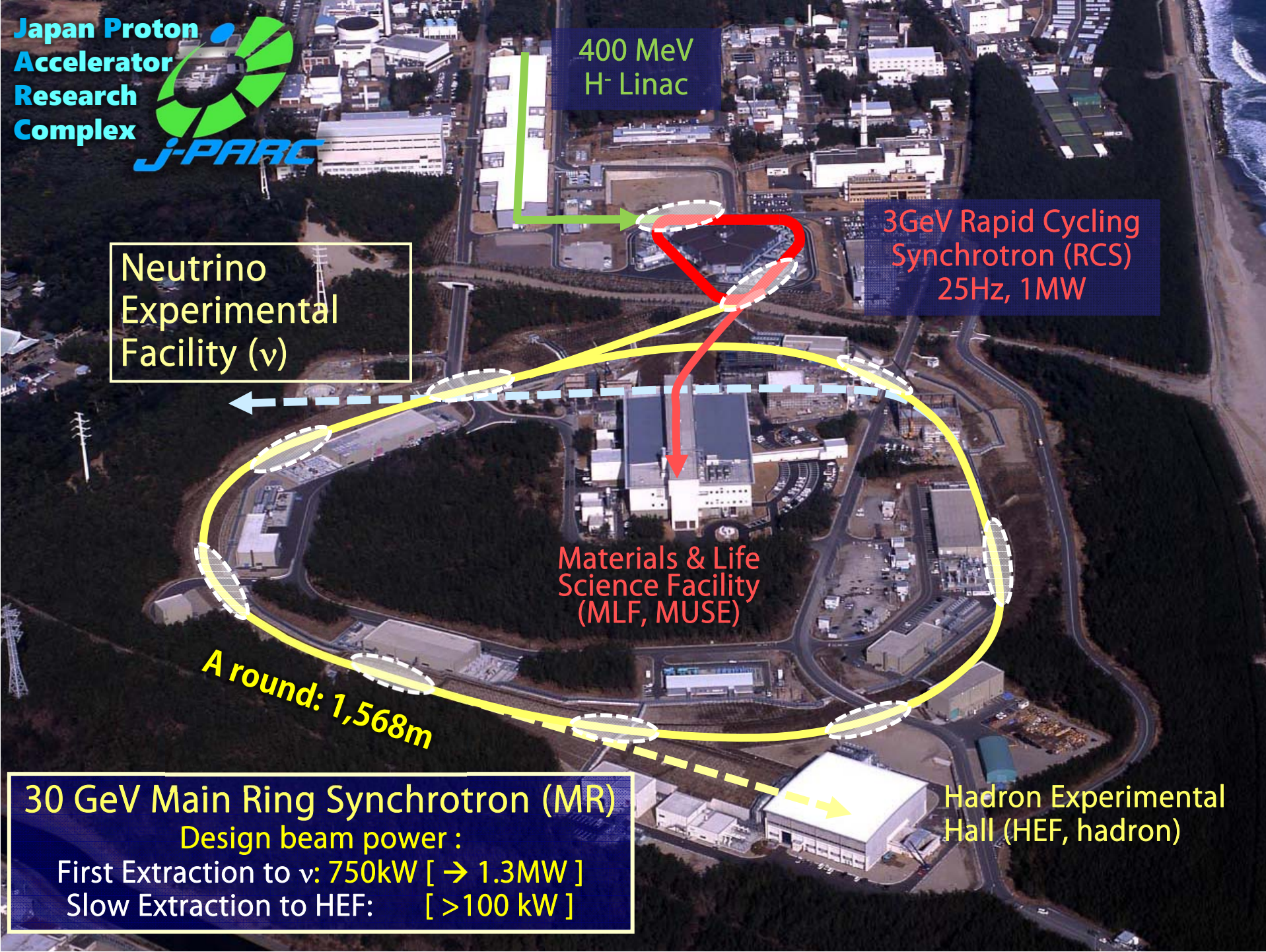
30 GeV Main Ring Synchrotron (MR)

Design beam power :

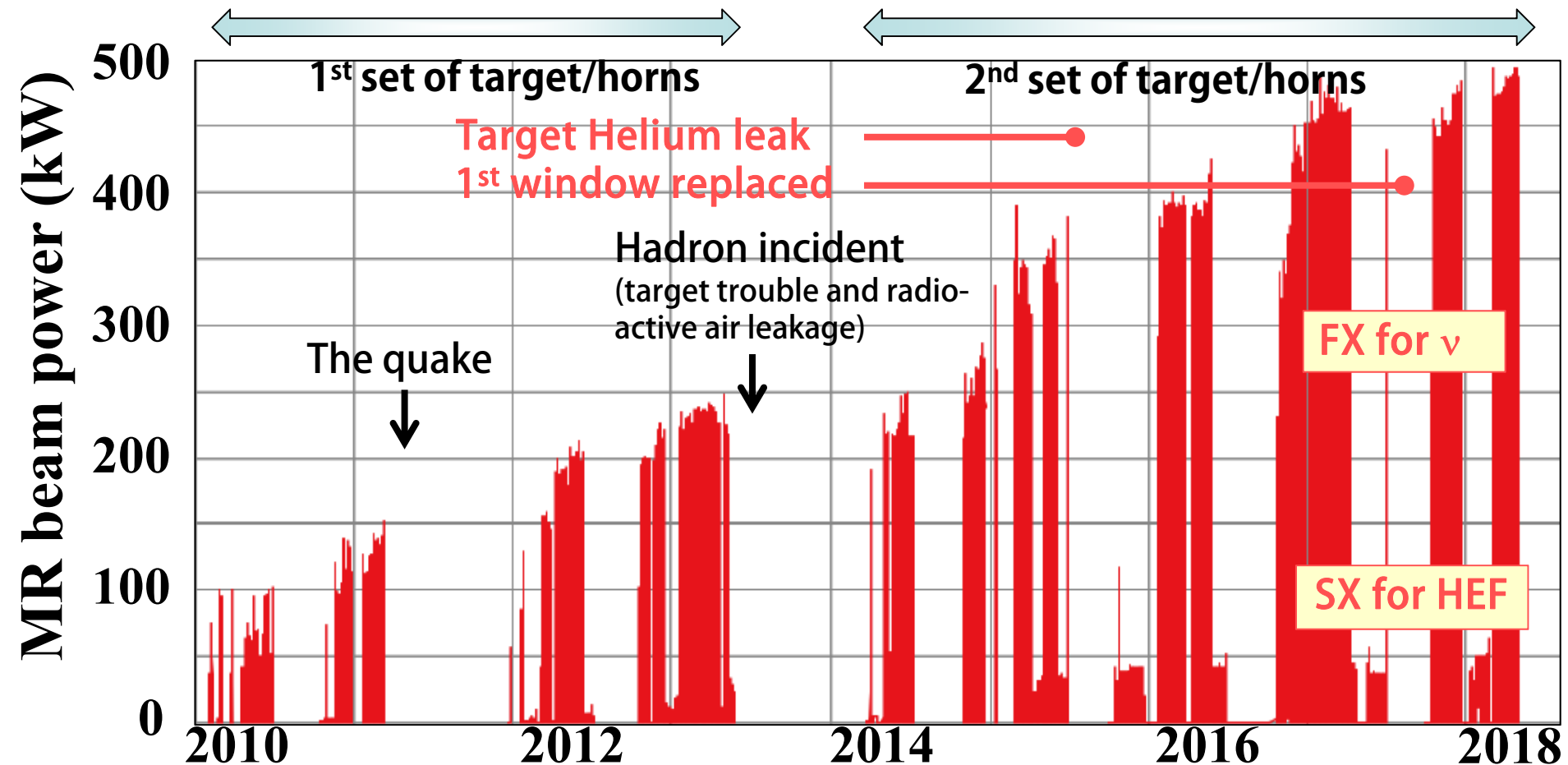
First Extraction to ν : 750kW [\rightarrow 1.3MW]

Slow Extraction to HEF: [>100 kW]

Hadron Experimental
Hall (HEF, hadron)

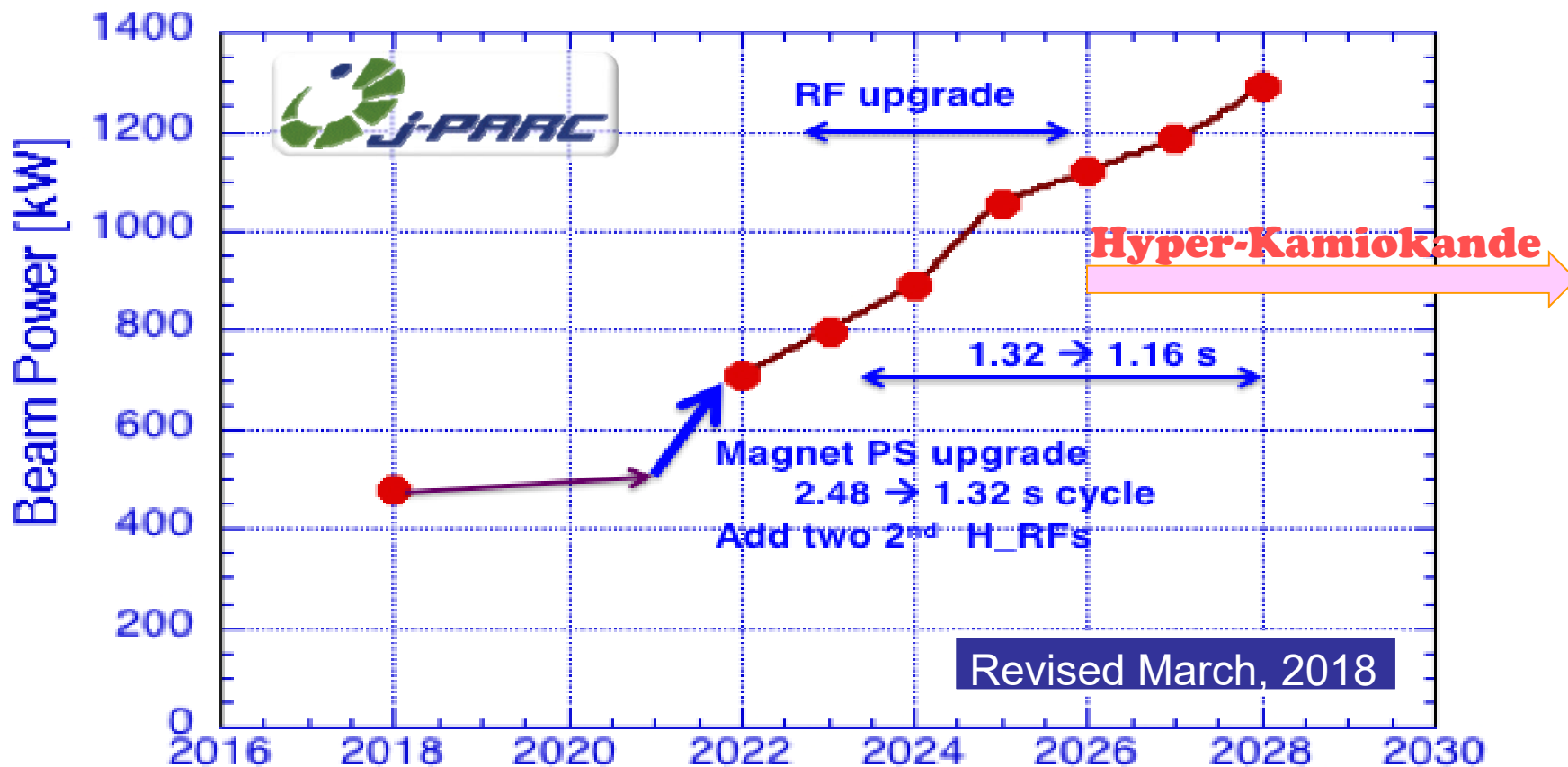


Operational History of Main Ring



- The 485 kW beam power has been achieved with 2.5×10^{14} ppp/2.48sec
- No major problems on neutrino facility target and beam-window
 - ◆ He leak at target outlet pipe → fixed by remote handling
 - ◆ The 1st beam window is replaced to (nearly) identical 2nd window

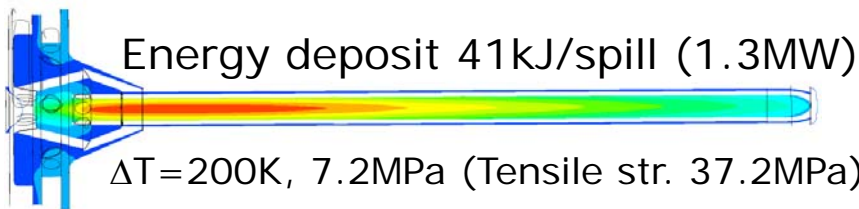
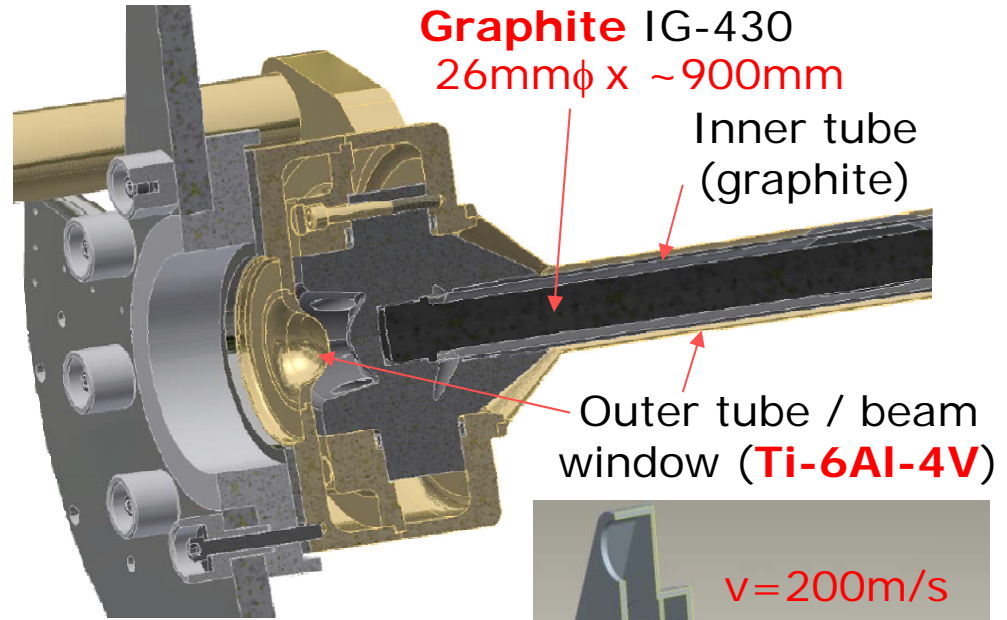
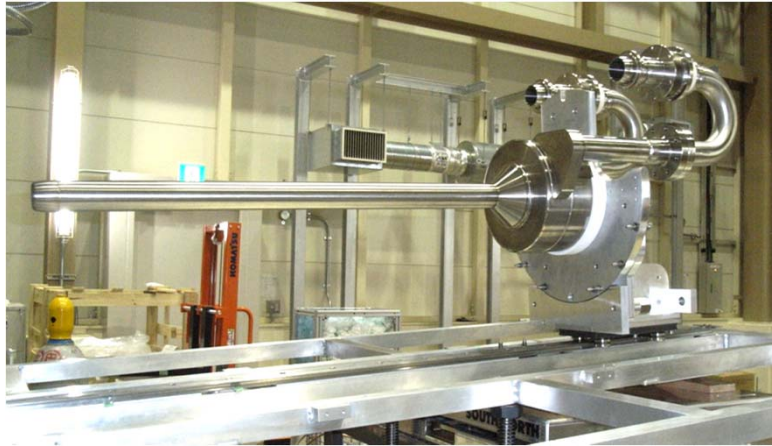
MR beyond 1 MW with Doubled Rep-rate



Beam power	485 kW (achieved)	750 kW (proposed)	1.3 MW (proposed)
Beam energy	30 GeV	30 GeV	30 GeV
Beam intensity (ppp)	2.5×10^{14}	2.0×10^{14}	3.2×10^{14}
Repetition cycle	2.48 s	1.32 s	1.16 s



1.3 MW Target Upgrade – Graphite & Ti

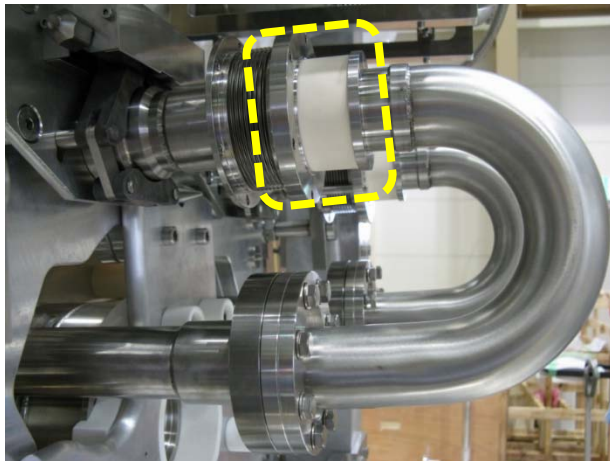


	0.75 MW	1.3 MW
Helium pressure	1.6 bar	5 bar
Pressure drop	0.83 bar	0.88 bar
Helium mass flow	32 g/s	60 g/s
Heat load	23.5 kW	40.8 kW
US window temp	105 ° C	157 ° C
DS window tem	120° C	130° C
Graphite Max. temp.	736° C	909° C

Lifetime 5years under 100ppm

- **High Temperature** : Oxidization of graphite will be the limiting factor on target lifetime
 - ◆ CVD-SiC coated graphite in BLIP/HiRadMat
- Radiation damage on Ti beam window under higher pressure

Leak at Helium Outlet Pipe & Remote Handling



- Failure of joint/ceramic from movement of stainless pipes (stress relieving)
- Thermal fatigue failure of the diffusion bonded joint/ceramic
- Remote Exchange in 2015
- **No leak since then**

(A.Atherthon HPTW16)

Diffusion bond
(Aluminum)

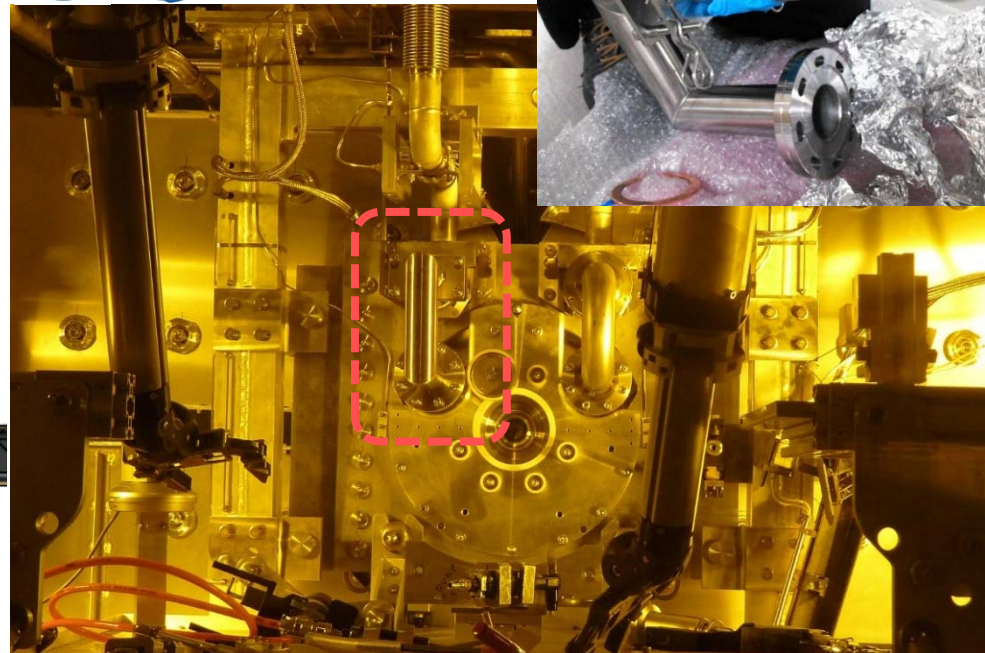
316L
Stainless

Garlock
flange

Bellows

304L
Stainless
Bent in
cold
process

Alumina ceramic

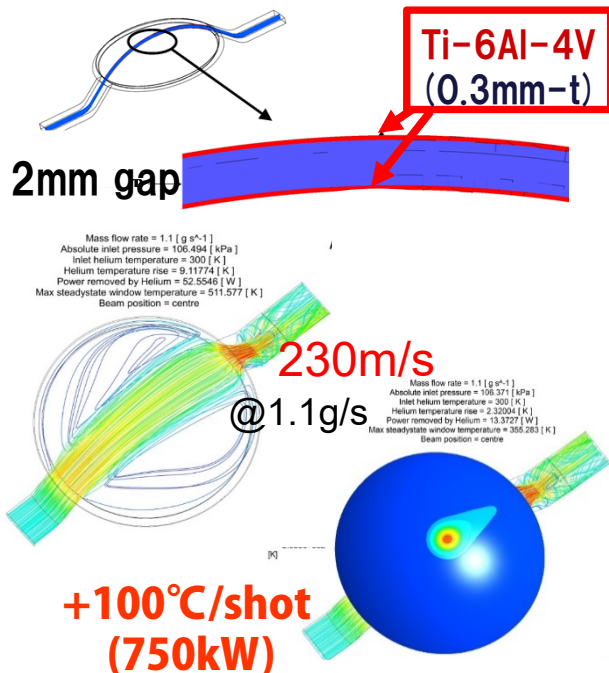




Ti-6Al-4V Beam Window



- Periodic thermal stress wave caused by the intense proton beam energy deposition
- 750kW operation will cause radiation damage of ~1DPA/ops-year, whereas significant irradiation hardening and loss of ductility has been reported with 0.3DPA (no higher DPA data exists)
- No known data exists on high cycle fatigue ($>10^3$ cycles) of irradiated titanium alloys



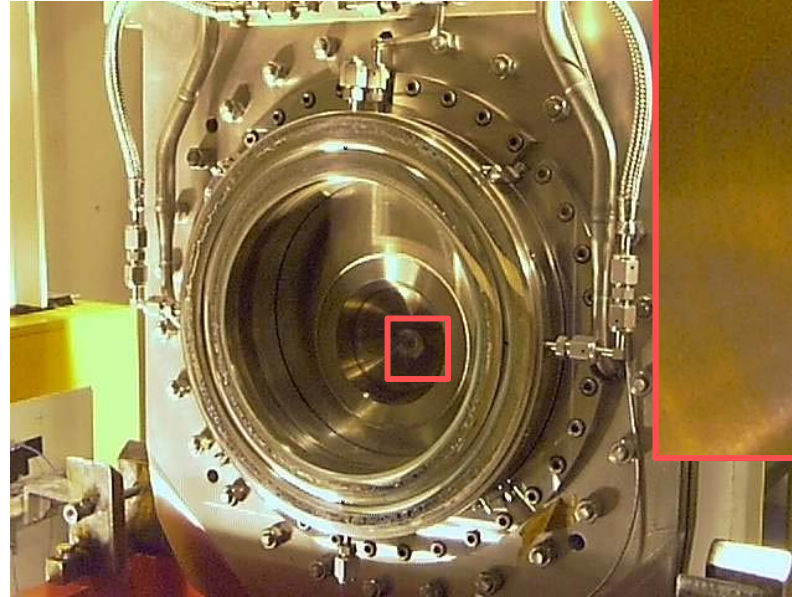
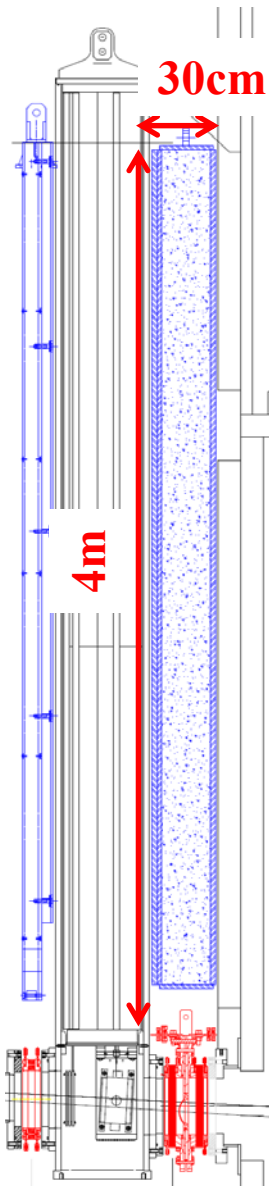
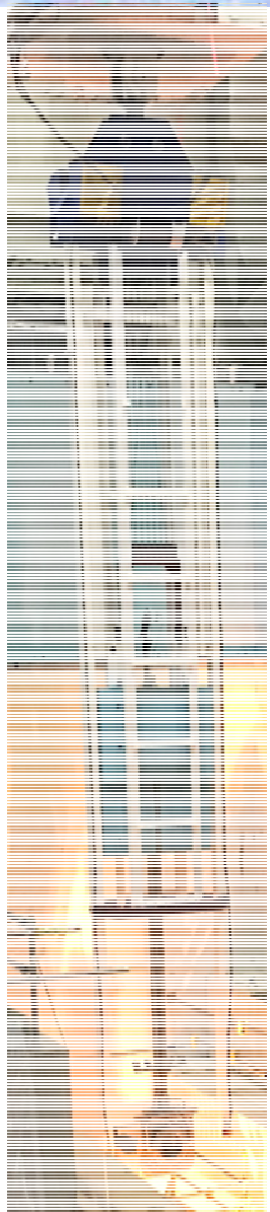
Beam Power	PPP	Rep. cycle	POT / 100 days
470kW (achieved)	2.4×10^{14}	2.48 sec	(0.8×10^{21})
750kW (proposed)	2.0×10^{14}	1.3 sec	1.3×10^{21}
750kW [original plan]	3.3×10^{14}	2.1 sec	1.3×10^{21}
1.3 MW (proposed)	3.2×10^{14}	1.16 sec	2.4×10^{21}

designed

~8M pulses/yr

~1DPA/yr (MARS)

Window Remote Maintenance (2017 summer)



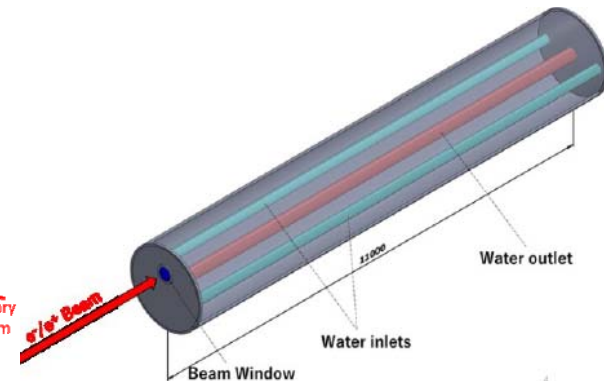
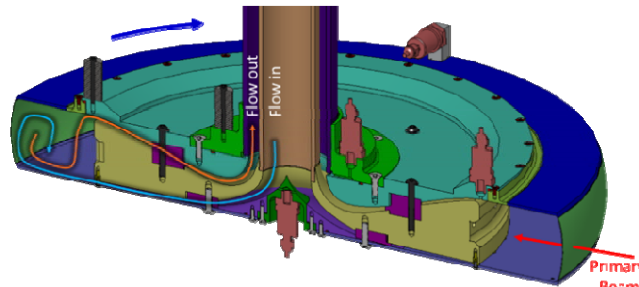
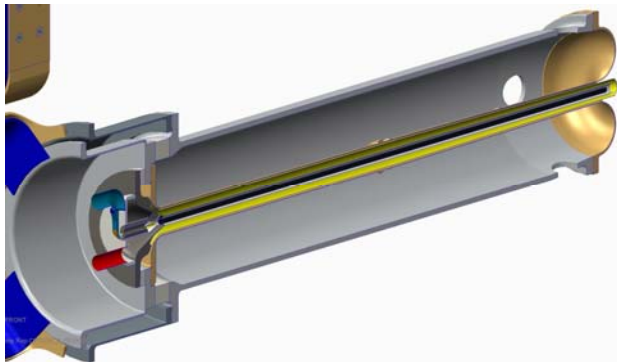
After 2.2×10^{21} pot
~2DPA(NRT)

- The 1st window survives to the 2.2×10^{21} POT comparable to 1.3MW/ops-yr
- Discoloration/deposit/damage of Ti-6Al-4V on TS vessel side visually identified
 - ◆ Helium in TS vessel was humid, and the damage can be from radiochemical reaction (accelerator vacuum side show less signatures)
 - ◆ Need to improve helium circulation system to remove the humidity
- We wish to perform PIE at JAEA's hot-lab, while it will take time to clear radiation safety regulations and license problems at J-PARC

Radiation damage studies on Ti-alloys



- Titanium alloy Ti-6Al-4V is widely adopted as a targetry material:
 - ◆ J-PARC neutrino primary beam window, target window & containment vessel
 - ◆ J-PARC hadron facility target chamber window
 - ◆ LBNF reference design target window and target containment vessel
 - ◆ MSU-FRIB Beam Dump
 - ◆ ILC 14MW main water dump beam window
- Relatively little known on how this Ti alloy is affected by high energy proton irradiation
- Imperative to research radiation damage effects to enable :
 - ◆ Accurate component lifetime prediction
 - ◆ Design of robust multi-MW components, and
 - ◆ Choice of better alloy or development of new materials to extend lifetimes



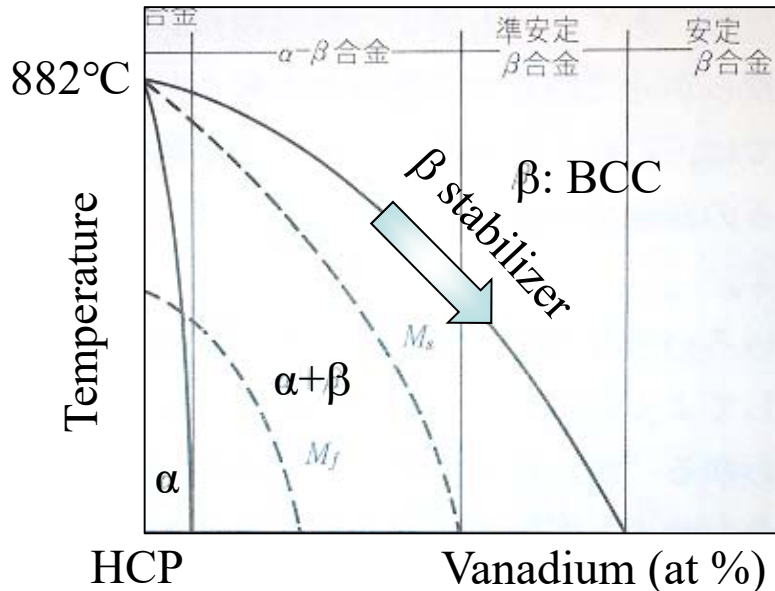
Classification of Ti-alloys & BLIP specimens



Comprehensive understanding on radiation damage effects for different Ti alloys

β and $\alpha+\beta$ alloy require proper heat treatment to reinforce strength by precipitation of alloy elements:

1. **Solution Treatment:** Keep proper high temp to solute more alloy element than RT
2. **Rapid Cooling** to RT keeping the condition
3. **Aging treatment:** Keep temp moderately higher than RT to precipitate the alloying components as fine intermetallic compound



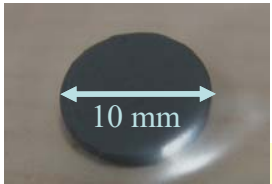
□ : Samples in BLIP capsules

組成	熱処理	引張性質		
		引張強さ (MPa)	耐力 (MPa)	伸び (%)
純チタン(α) Comercially Pure Ti				
JIS 1種 Gr1(α)	A	270~410	≥ 165	≥ 27
JIS 2種 Gr2(α)	A	340~510	≥ 215	≥ 23
JIS 3種	A	480~620	≥ 345	≥ 18
JIS 4種	A	550~750	≥ 485	≥ 15
α 合金				
Ti-5 Al-2.5 Sn Gr6(α)	A	862	804	16
α リッチ α-β 合金				
Ti-8 Al-1 Mo-1 V	A	1 000	951	15
Ti-6 Al-2 Sn-4 Zr-2 Mo	A	980	892	15
α-β 合金				
Ti-3 Al-2.5 V Gr9($\alpha+\beta$)	A	686	588	20
Ti-6 Al-4 V	A	980	921	14
Gr5/23($\alpha+\beta$), α'-UFG	STA	1 170	1 100	10
Ti-6 Al-6 V-2 Sn	A	1 060	990	14
	STA	1 270	1 170	10
Ti-6 Al-2 Sn-4 Zr-6 Mo	STA	1 270	1 180	10
Ti-10 V-2 Fe-3 Al	STA	1 270	1 200	10
β 合金 Meta-stable β				
Ti-13 V-11 Cr-3 Al	STA	1 220	1 170	8
Ti-3 Al-8 V-6 Cr-4 Mo-4 Zr	STA	1 440	1 370	7
Ti-11.5 Mo-6 Zr-4.5 Sn	STA	1 380	1 310	11
Ti-15 Mo-5 Zr-3 Al	STA	1 470	1 450	13
Ti-15 V-3 Cr-3 Al-3 Sn	STA	1 230	1 110	10

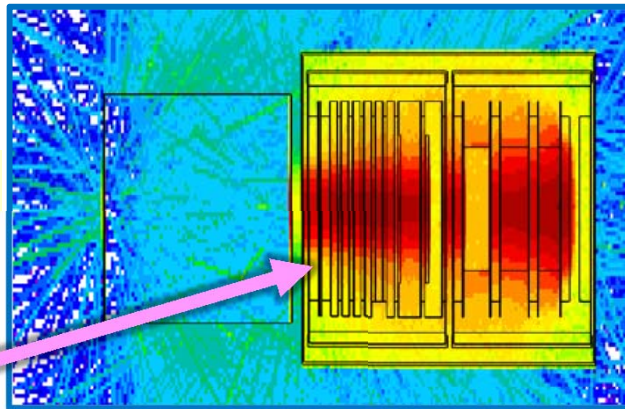
15-3Ti(β)

RaDIATE Irradiation Runs at BNL-BLIP Facility

SiC-coated graphite



→ D.Senor

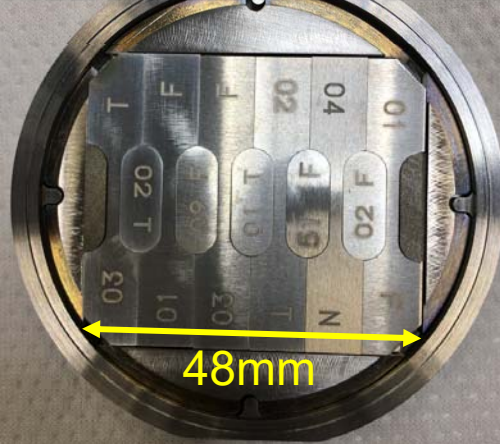


Accumulated Damage
Ti : 1.52 DPA peak (NRT)
0.93 DPA peak (Stoller)

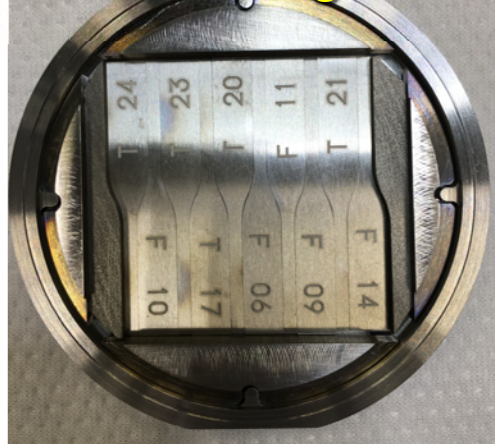


- 1st phase irradiation (2017)
 - ◆ Total POT: 1.76×10^{21} in 22 days @ 146μA average
 - ◆ Capsules shipped to PNNL
 - ◆ Tensile testing → A.Casella
 - ◆ 2 capsules newly installed
- 2nd phase irradiation (Jan-Mar 2018)
 - ◆ 2.81×10^{21} in 33 days @ 158uA average
 - ◆ Capsules to be shipped to PNNL soon.

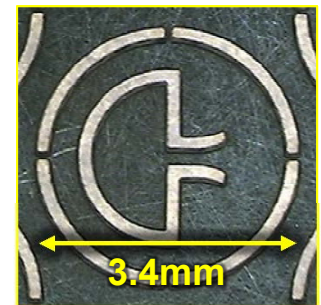
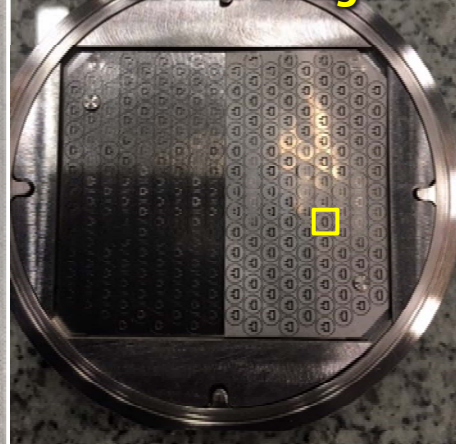
Macro-Tensile & Microstr.



Macro-Fatigue



Mesoscale-Fatigue

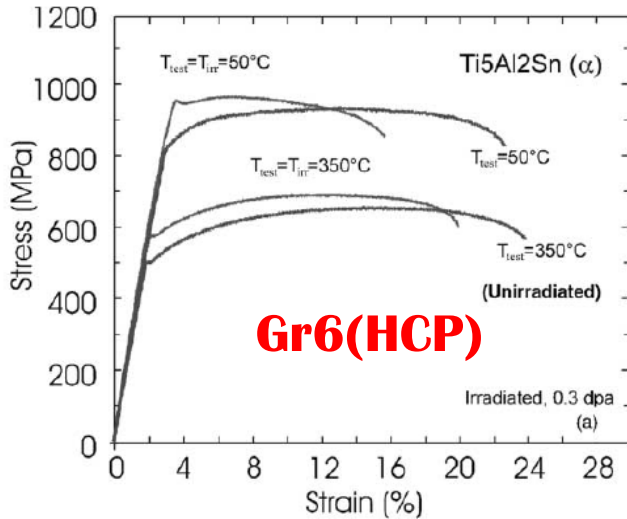


Damage-tolerant Candidates in BLIP Irradiation



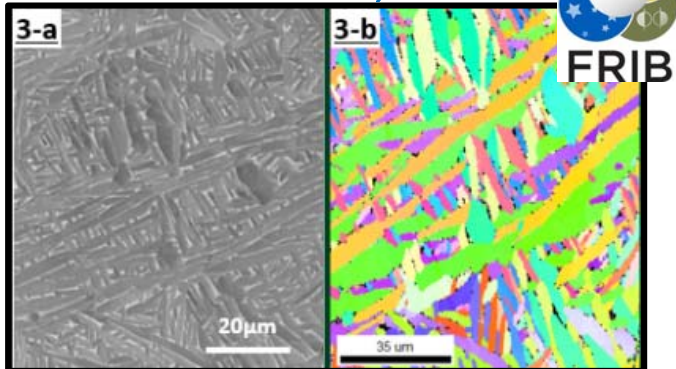
HCP α alloy Gr6

Better ductility (n 0.3DPA)



S. Tähtinen et al., JNM 307-311 (2002) 416

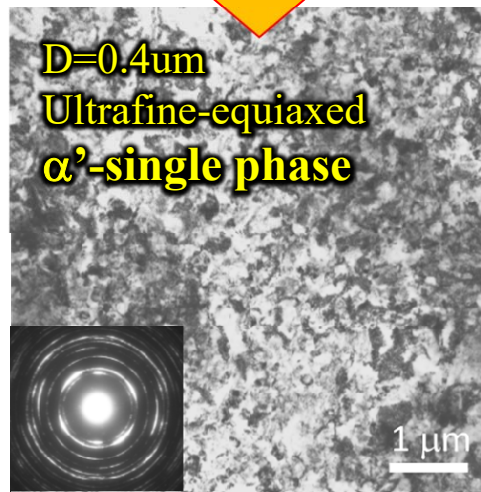
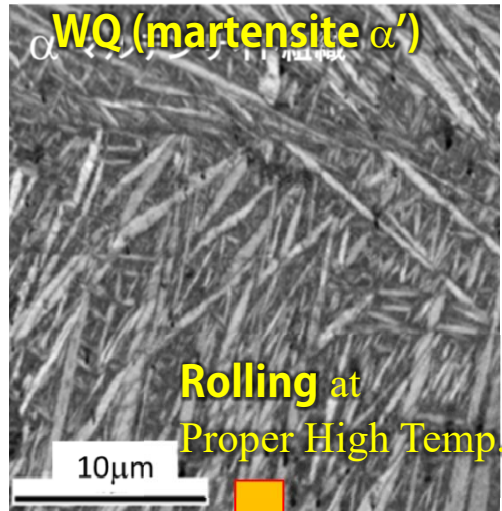
3D printing (Direct Metal Laser Sintered) Gr-23



F.Pellemoine, NBI2017+RaDIATE

64Ti α' -Ultra FineGrain

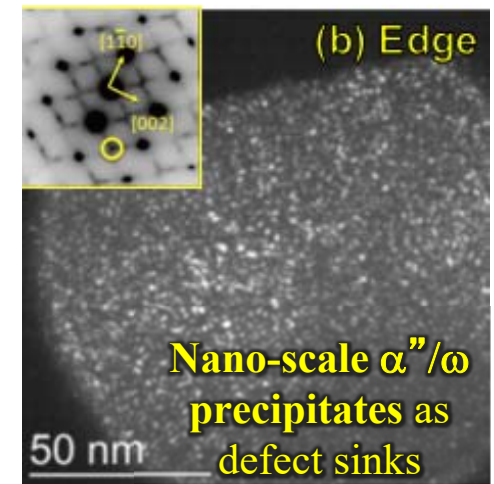
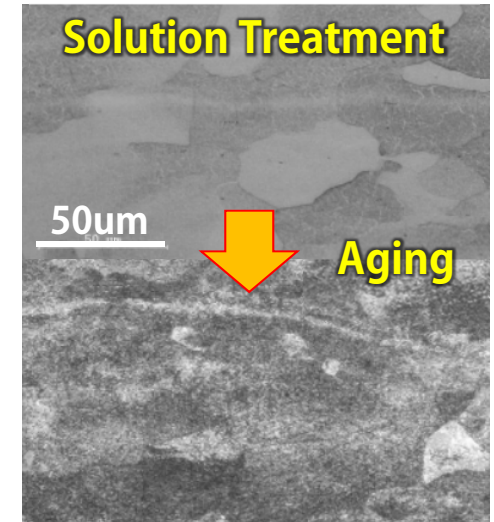
Rich grain boundaries



H.Matsumoto et al., Adv.Eng.Mat.13 (2011) 470

Metastable β 15-3Ti

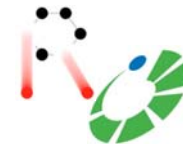
Nanoscale precipitates



T.Ishida et al, NME (2018) in press

→ D.Senor

High-Cycle Fatigue Data on Ti alloys to be Available



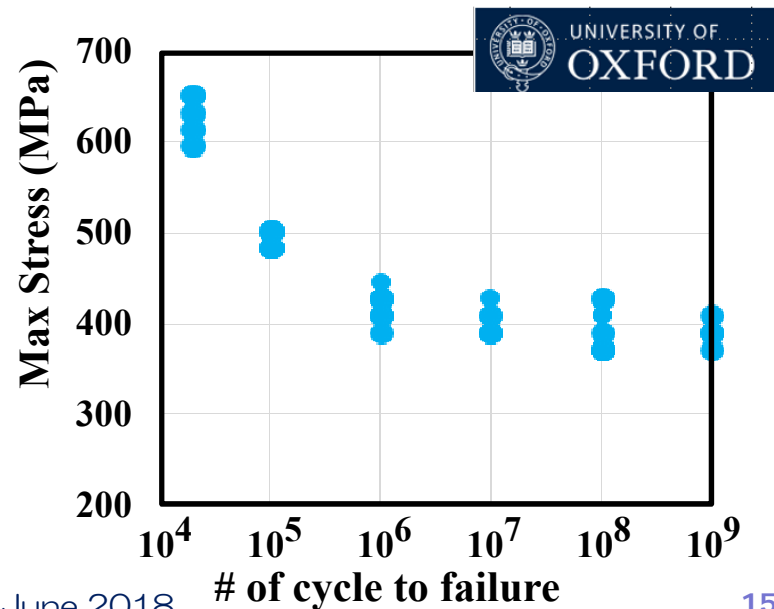
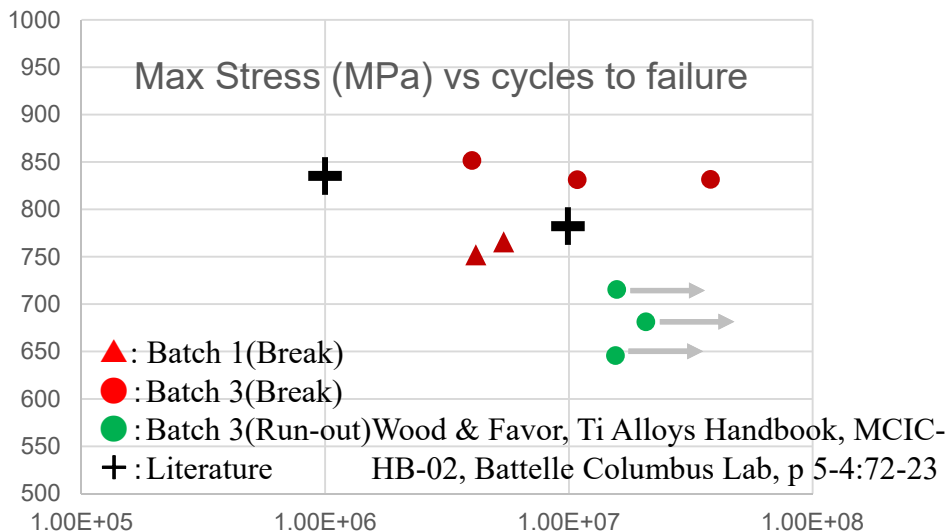
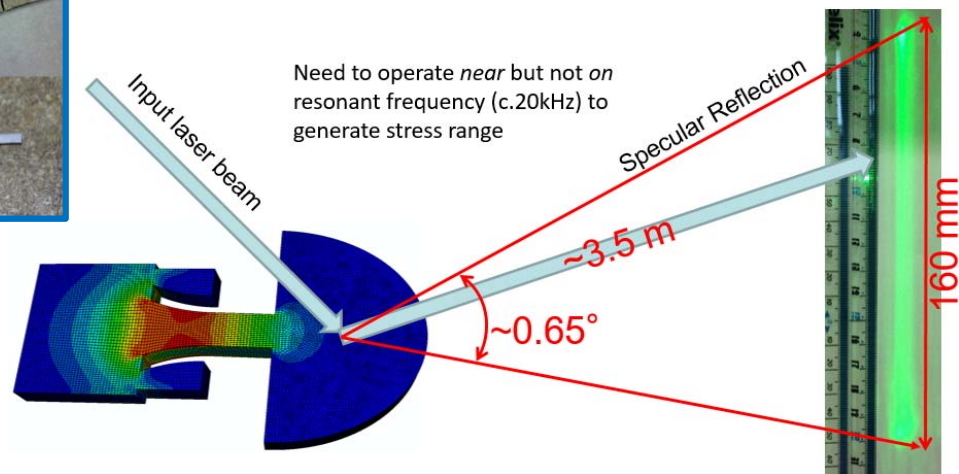
Macro-scale Fatigue Testing

■ Gr5/23



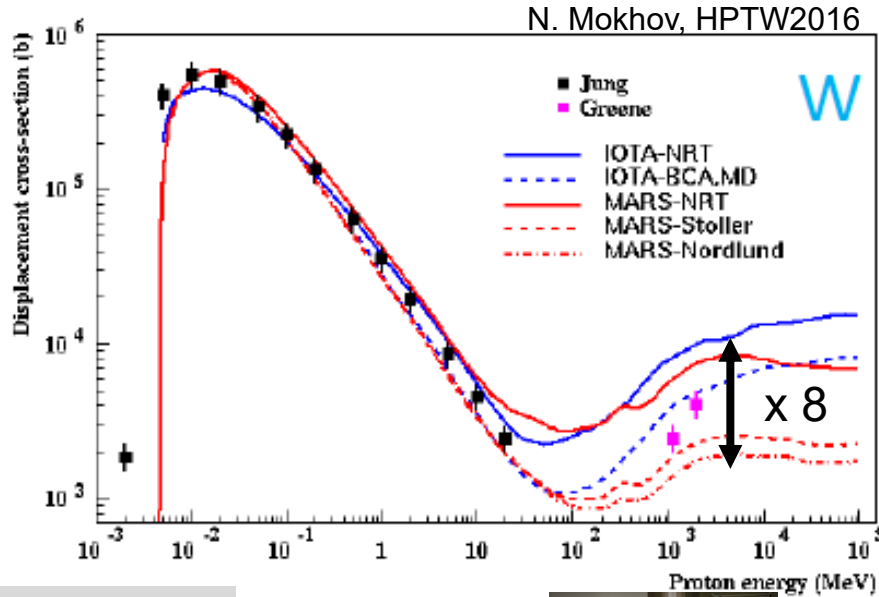
Meso-scale Fatigue Testing

■ Gr23 A&STA, Gr2, 15-3Ti



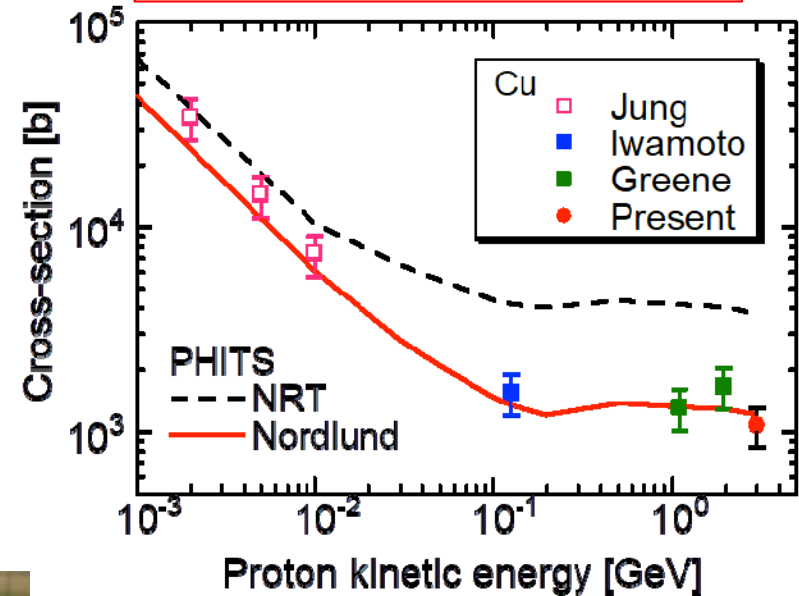
Measurement of displacement cross-section for 3-GeV proton at J-PARC

S.Meigo et al., IPAC2018, MOPML045



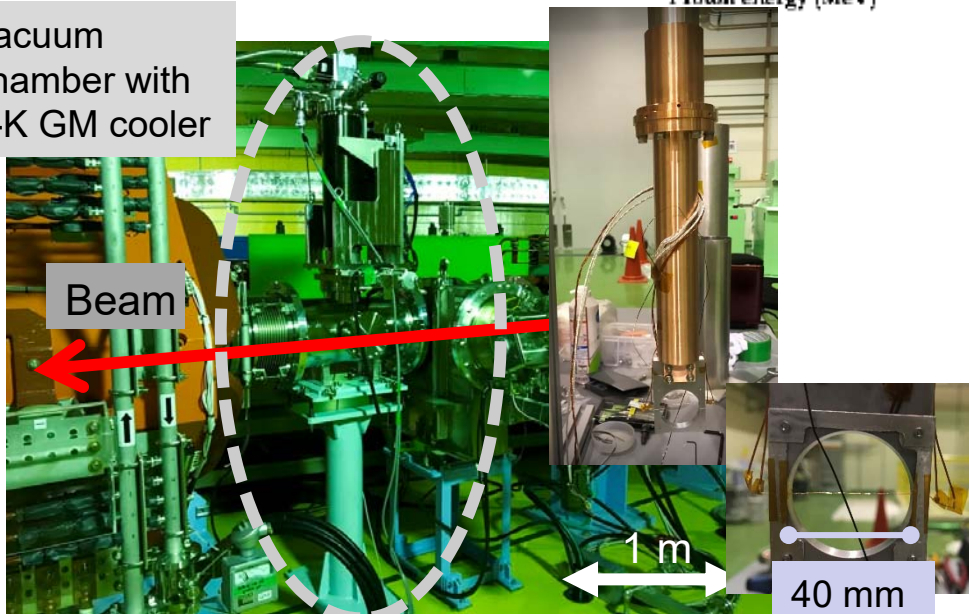
- Experiment at 3-GeV Rapid Cycling Synchrotron (RCS)
- Under cryotemperature (~ 20 K), displacement cross section (σ) was obtained by increase of resistivity ($\Delta\rho_{Cu}$) due to proton irradiation with average flux ($\overline{\phi(E)}$)

$$\sigma_{exp}(E) = \Delta\rho_{Cu} / (\overline{\phi(E)} \rho_{FP}),$$



NRT overestimates about 4 times of the present data, while Nordlund model drastically improves.

Vacuum chamber with 4-K GM cooler



Summary



- J-PARC Neutrino target/window : designed with 3.3×10^{14} ppp, hopefully applicable for coming 750kW→1.3MW operation with minor upgrades
 - ◆ Oxidization of graphite will determine the life of target. SiC-coated graphite specimen provided for BLIP irradiation / HiRadMat test
 - ◆ Beam window survived after 2.2×10^{21} pot. Need PIE to evaluate damage while it requires time to clear rad.safety regulations & license problems
- Study on radiation damage effect of Titanium alloys with a few DPA region is critical to determine/improve service life of targetry applications
 - ◆ BLIP irradiation run will provide data upto 1.5DPA (NRT)
 - ◆ 1st HCF data with both macro-bend and mesoscale tests
 - ◆ Damage-tolerant candidates: Gr-6/Gr-23 DMLS/Gr-5 α' -UFG/15-3Ti
 - ◆ Comprehensive understanding of radiation damage effect on different type of Titanium alloys and different heat treatments will be obtained.
- Displacement xsec. measurement for 3GeV proton carried out at J-PARC.
 - ◆ NRT overestimates about 4 times. Nordlund model agrees.
 - ◆ 30GeV at J-PARC MR approved. Collab. with Fermilab/ CERN underway.