

Microstructural Characterization of Proton-Irradiated Ti-15V-3Cr-3Sn-3Al and SiC-Coated Graphite

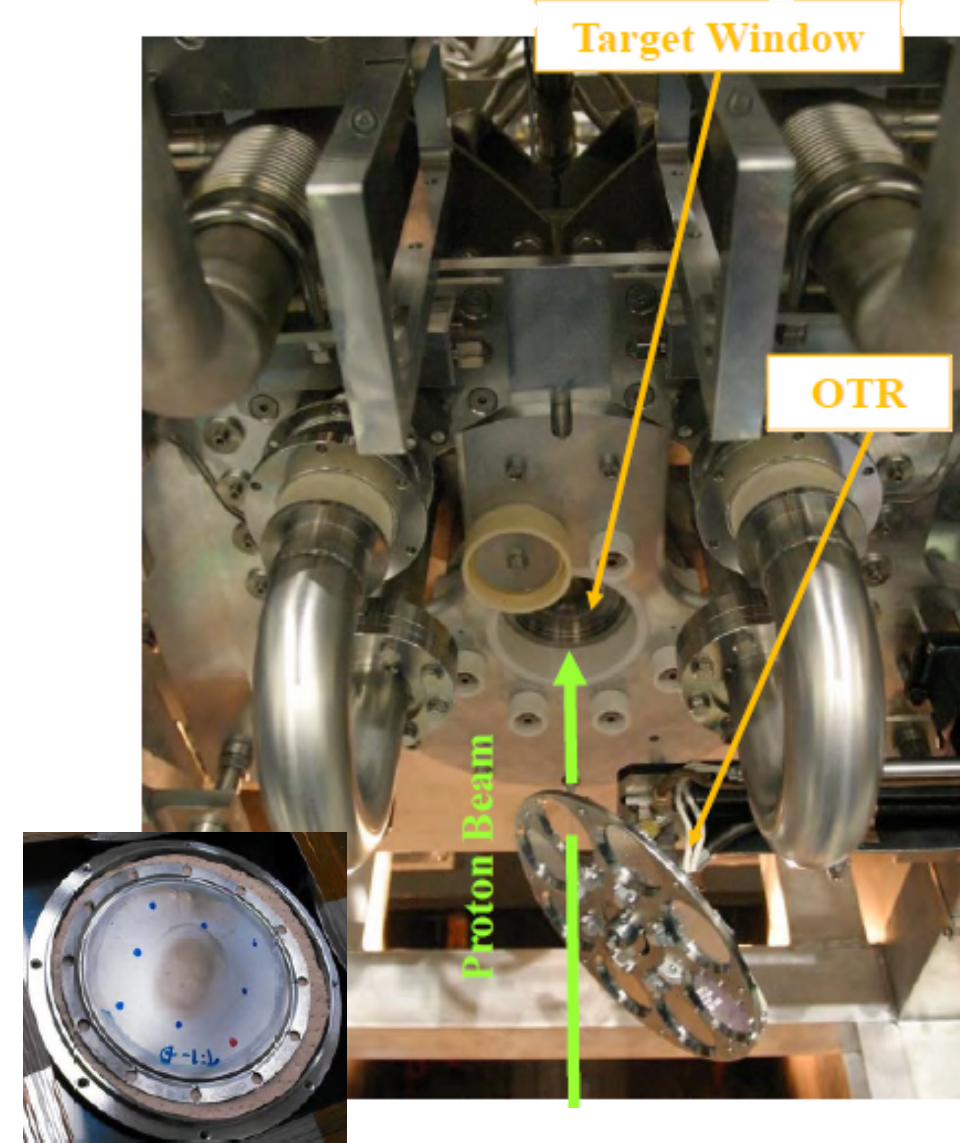
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Optical Transition Radiation (OTR) Monitors

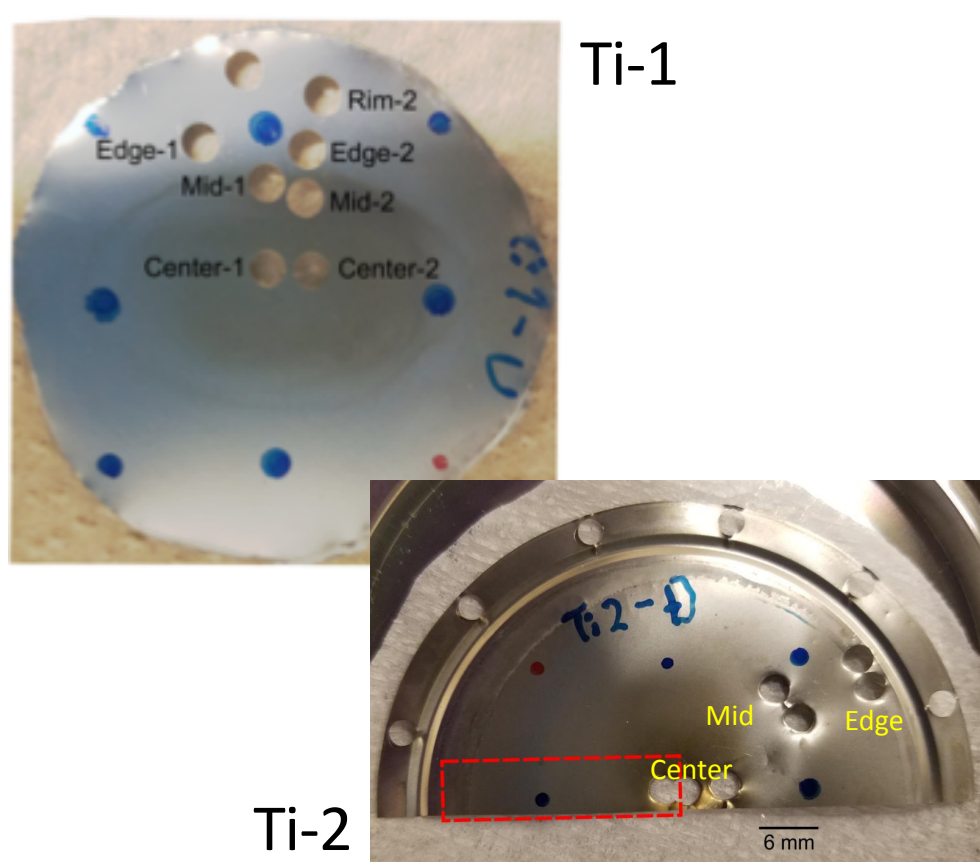
Background: The OTR foil, made of metastable β -Ti alloy about 50 μm thick, indicates proton beam shape and alignment

Motivation: After exposure to the 30 GeV proton beam at 100-130°C, the area of the foil illuminated by the beam became discolored, with reduced sensitivity

Objective: To understand if radiation damage or other effects caused the foil degradation

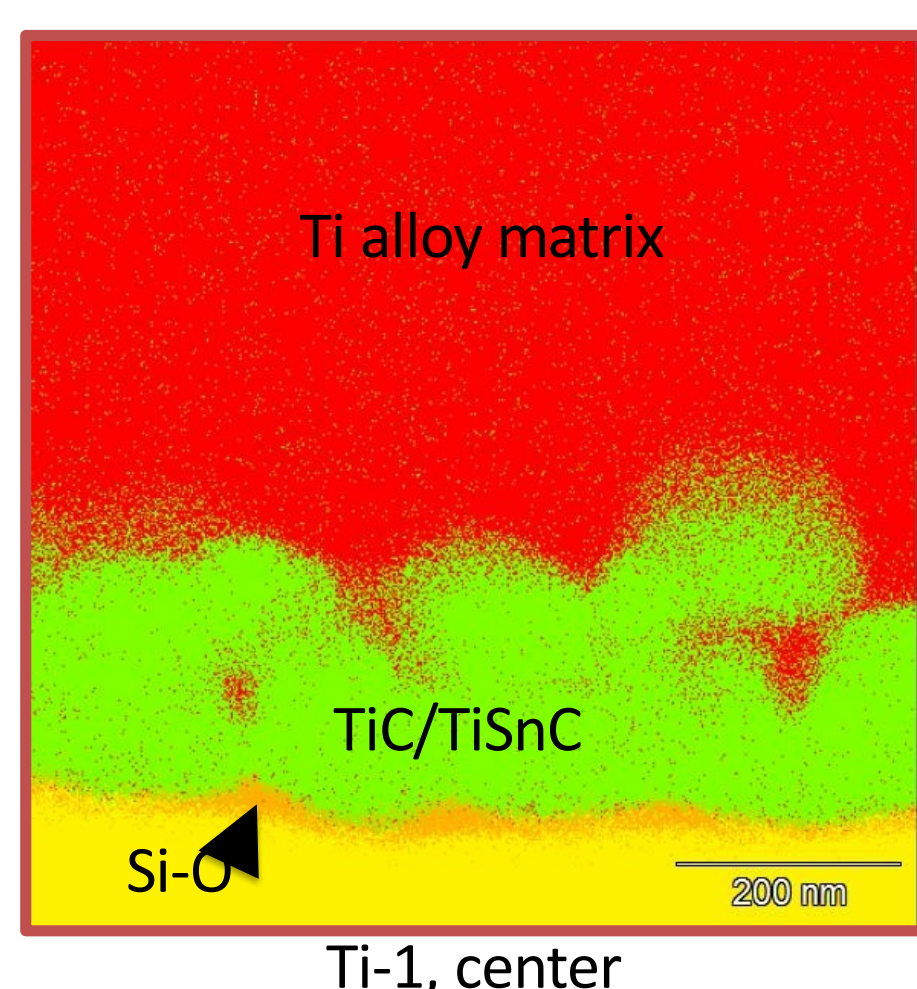
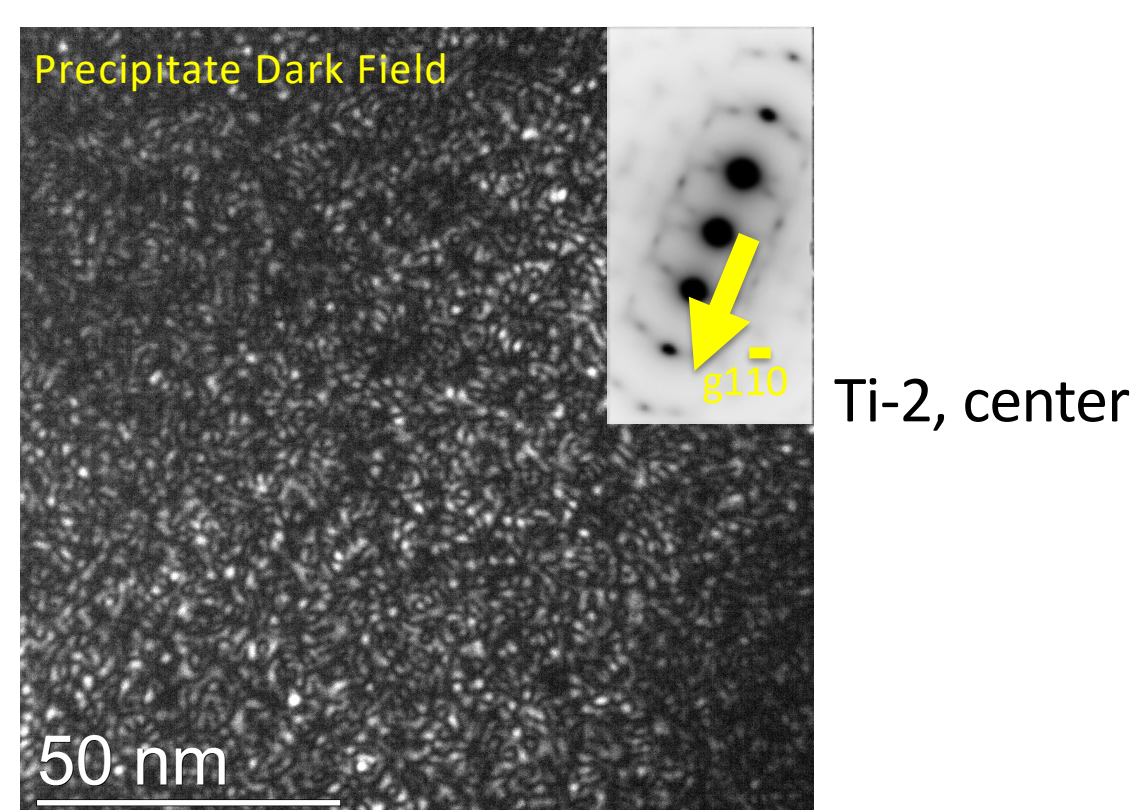
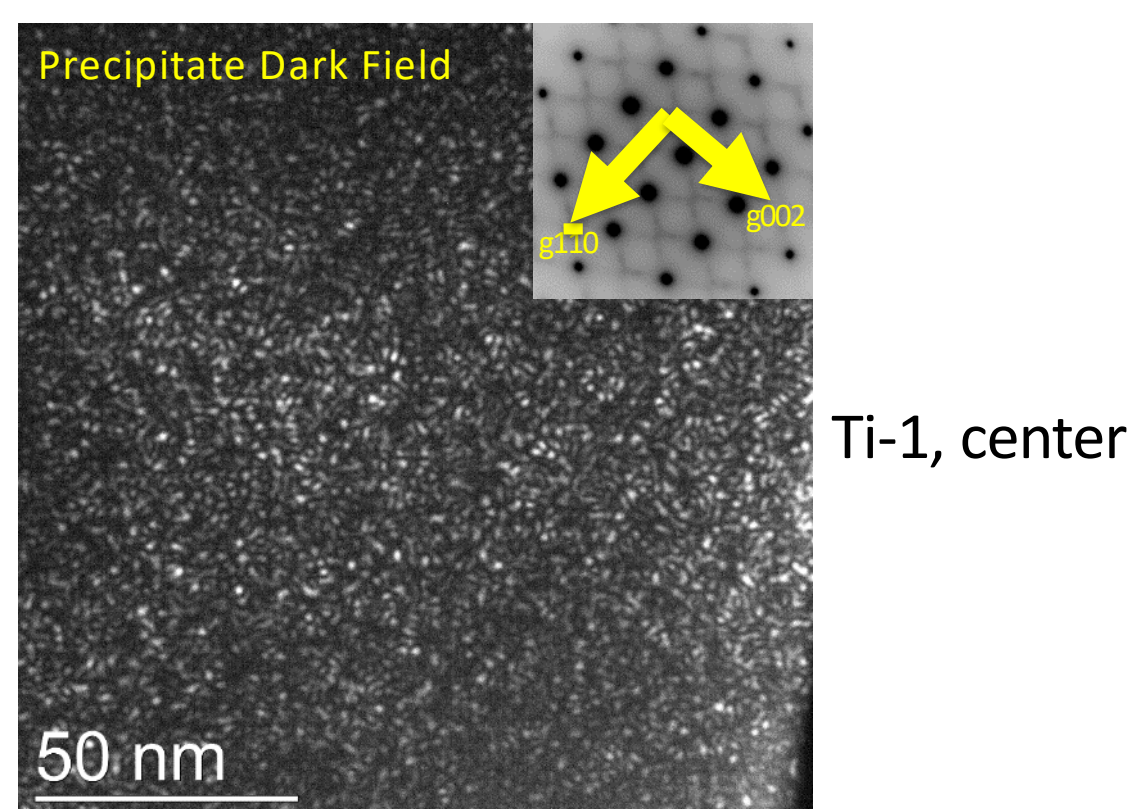


Transmission Electron Microscopy



- Three OTR foils studied
 - Unirradiated archive
 - Ti-1 (1.4×10^{20} pot, ~ 0.1 dpa)
 - Ti-2 (5.2×10^{20} pot, ~ 0.3 dpa)
- TEM samples prepared from 3 mm disks punched from OTR foils
- Disks electrochemically etched in a Tenupol jet polisher

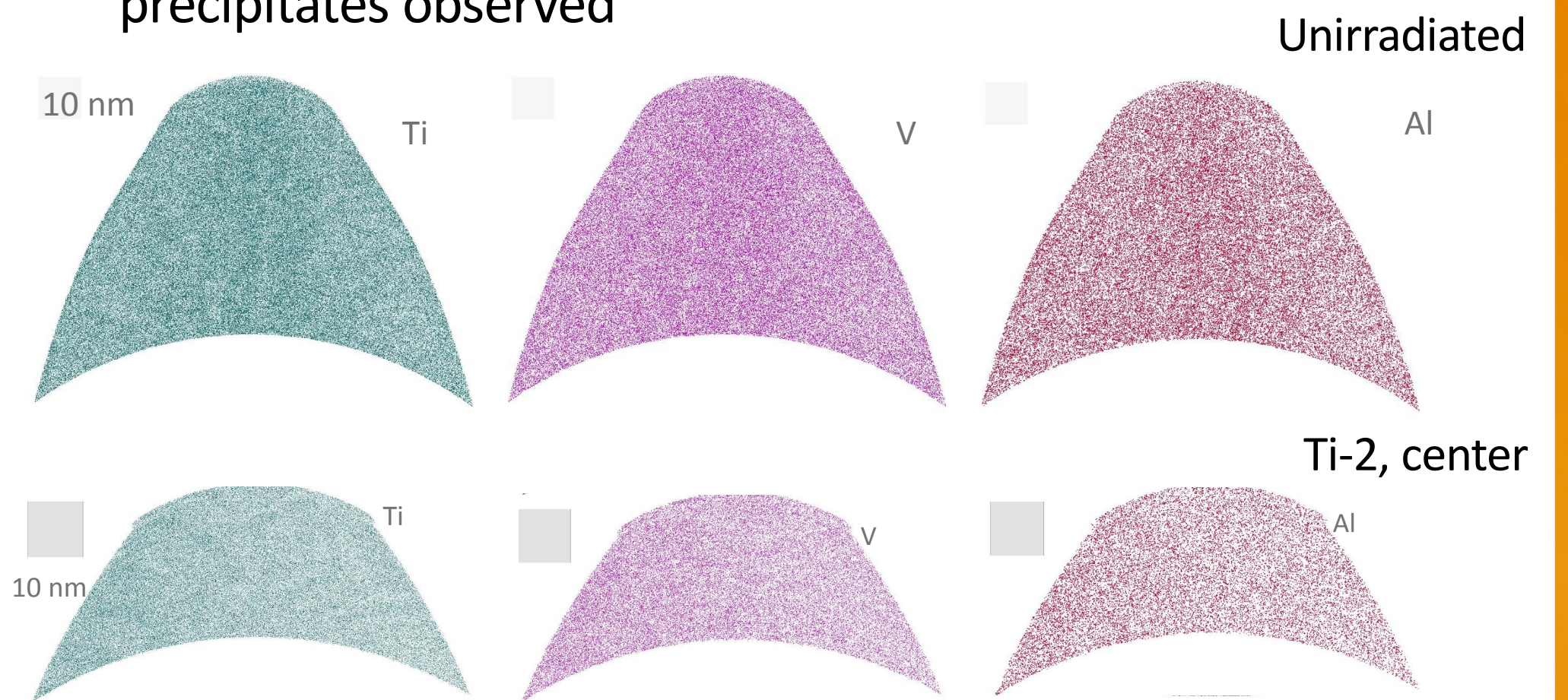
- Dark field images centered on one of the diffuse streaks in the diffraction pattern shows typical precipitates in Ti-1 and Ti-2
- Believed to be fine w-phase precipitates of 1-2 nm in size
- Comparable to precipitates observed in unirradiated archive foil and TEM samples taken near the edge of Ti-1 and Ti-2, thus not the result of irradiation damage
- Any incipient radiation damage must be at or below resolution limit



- A combination of elemental mapping via energy dispersive x-ray spectroscopy (EDS), principal component analysis (PCA) and precession electron diffraction (PED) identified the composition of the surface discoloration
- The Si and C impurities on the surface were attributed to backstreaming diffusion pump oil, not radiation damage

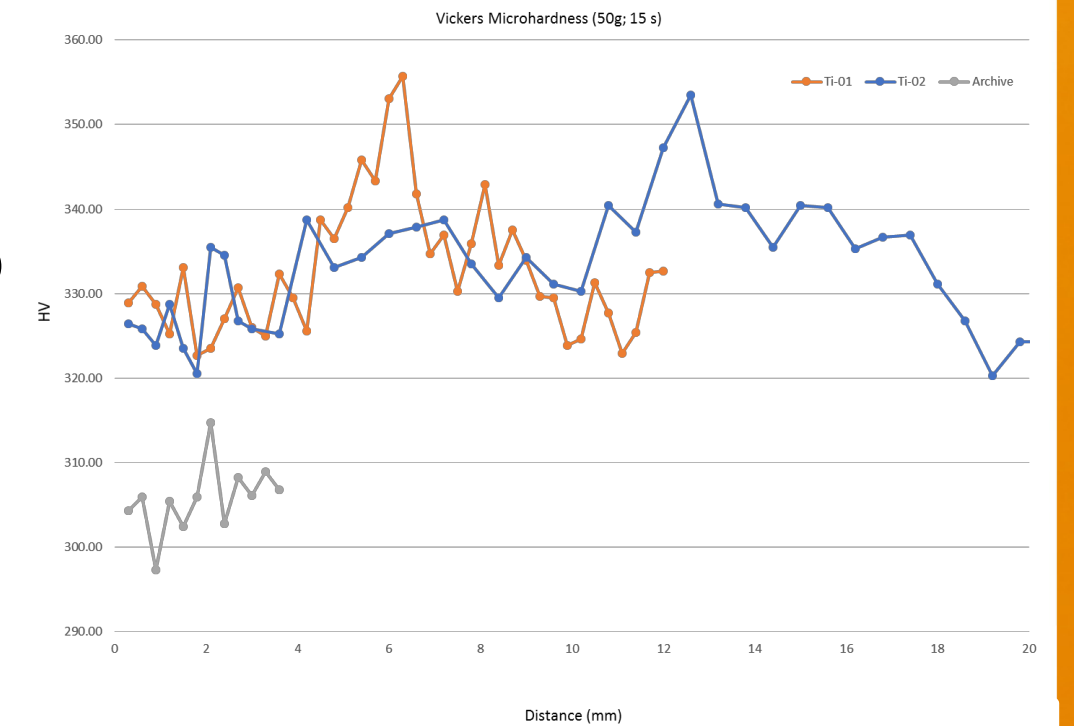
Atom Probe Tomography

- No obvious difference between APT results for unirradiated and irradiated OTR foils
- No statistically significant elemental segregation or precipitates observed



Microhardness

- Results suggest a $\sim 10\%$ increase in hardness for the irradiated foils compared to the unirradiated archive
- Hardness increase is uniform across foil and does not appear to be due to irradiation damage

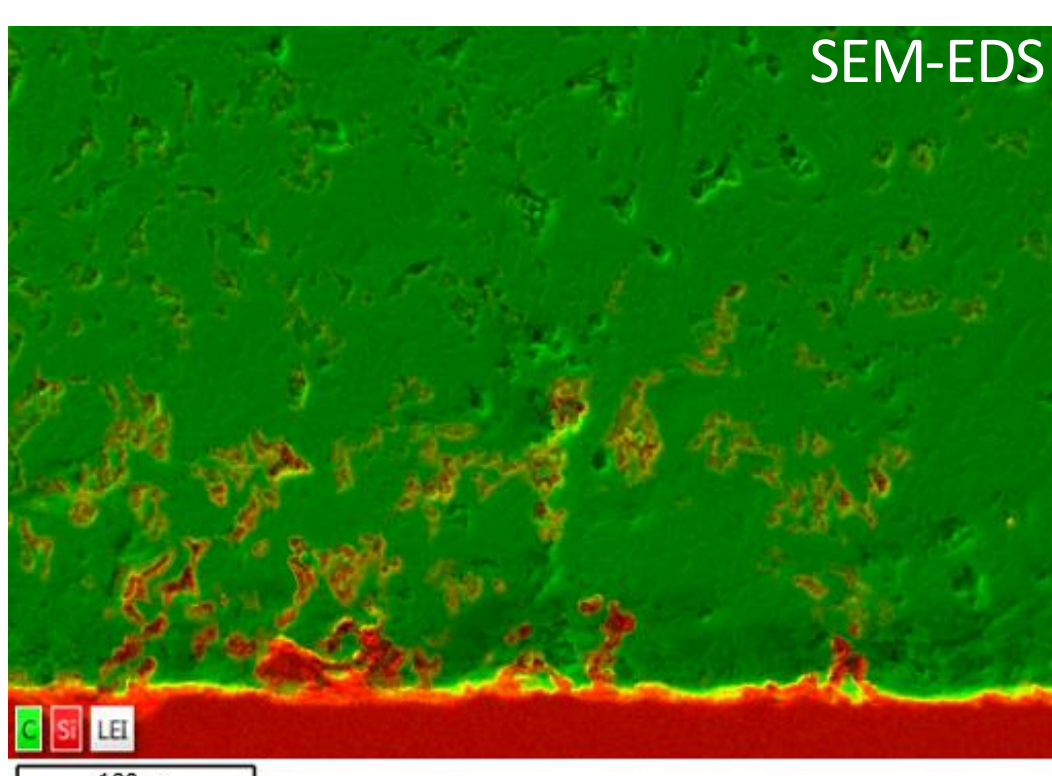
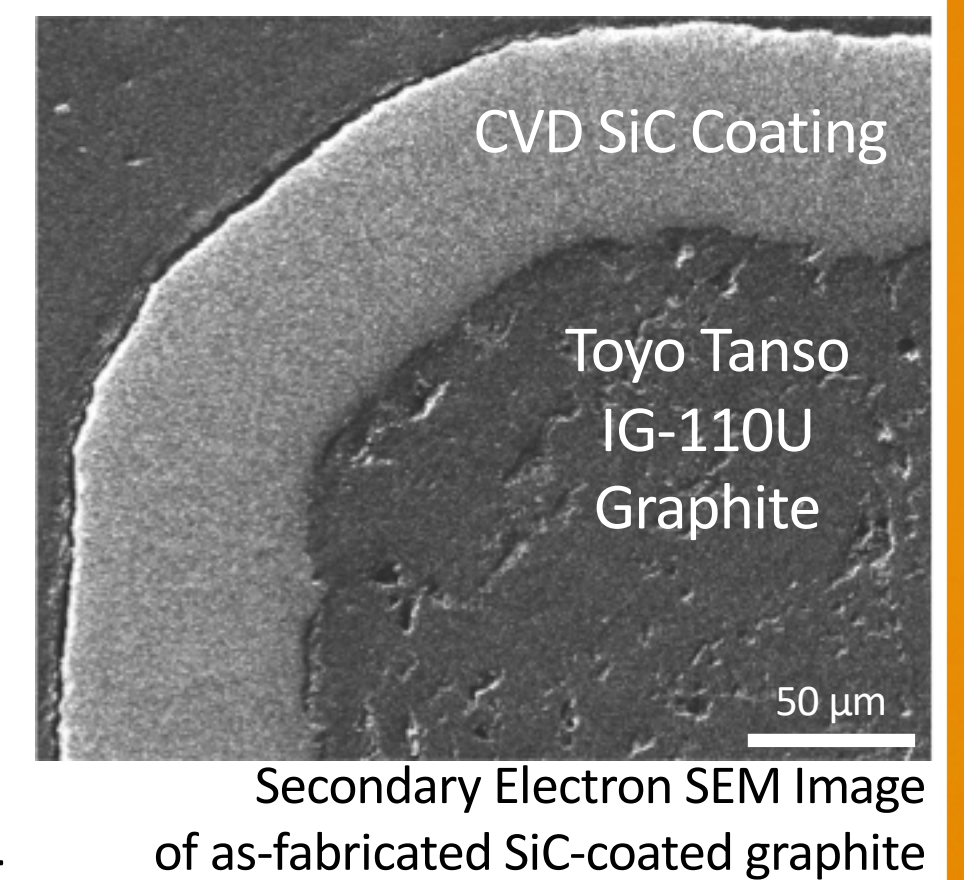


CVD SiC-Coated Graphite

Background: Graphite is an excellent pion/muon target but is susceptible to oxidation under operating conditions

Motivation: The chemical vapor deposited (CVD) SiC coating has the potential to improve oxidation resistance without significantly degrading physics performance

Objective: Determine if SiC coating retains integrity after irradiation



- As-fabricated SiC coating is dense and has good adhesion
- Some ingress of SiC into porous graphite during CVD
- Samples irradiated by 160 MeV protons at 240°C and 0.05 dpa
- Irradiated samples successfully extracted from capsule
- SEM microscopy on irradiated samples to evaluate condition of SiC coating is pending

ABOUT

Pacific Northwest National Laboratory

The Pacific Northwest National Laboratory, located in southeastern Washington State, is a U.S. Department of Energy Office of Science laboratory that solves complex problems in energy, national security, and the environment, and advances scientific frontiers in the chemical, biological, materials, environmental, and computational sciences. The Laboratory employs nearly 5,000 staff members, has an annual budget in excess of \$1 billion, and has been managed by Ohio-based Battelle since 1965.

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