OVERVIEW OF MEMBERS STATES' AND IAEA ACTIVITIES IN THE FIELD OF ACCELERATOR DRIVEN SYSTEMS (ADS)

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Outline

- Why ADS?
 - Background
 - Advanced nuclear fuel cycles
- □ ADS technology
- Member States activities
- □ IAEA activities
- Conclusions



- ☐ Worldwide significant nuclear energy figures
 - 372 GWe capacity distributed roughly 1/3 in each region USA / Europe / rest of the world
 - ~11'000 t HM annual spent fuel discharge
 - ~4'000 t HM industrial reprocessing capacity
 - ~130'000 t HM cum stored spent fuel inventory
 - ~70'000 t HM reprocessed and transformed into HLW and spent LWR-MOX



- Repository needs and characteristics
 - Present worldwide spent fuel and HLW arising would need TWO and ONE Yucca Mountain size repositories, respectively
 - Spent fuel repository: high Pu content ->
 non proliferation and criticality concerns
 - Spent fuel and HLW repository heat load determined by medium-lived fission products (137Cs and 90Sr)
 - Spent fuel and H. W repository radiotoxicity determined by minor actinides (Np, Am, Cm)



- □ Spent fuel and HLW repository hazard vs. risk
 - Partitioning and Transmutation (P&T)
 objective → reduction of long-term hazard of
 spent fuel or HLW repository by
 transforming long-lived radionuclides into
 short-lived or inactive elements
 - "Conventional" waste management objective
 reduction of long-term radiological risk
 (combination of potential hazard and confining properties)



- Hazard reduction (P&T objective) requires very different and much more fundamental measures as compared to <u>risk</u> reduction:
 - Long-term hazard of spent fuel and HLW is associated with the radioactive source, i.e. the transuranics
 - Short and long-term <u>risks</u> are due to the **mobility** of fission products in the **geosphere** and the possibility to **enter** the **biosphere**



- Nuclear fuel cycle options
 - Conventional
 - ✓ Once through fuel cycle with direct disposal of spent fuel (OTC)
 - ✓ Aqueous reprocessing fuel cycle with vitrification of high-level liquid waste (RFC)
 - Advanced fuel cycle with partitioning of actinides (AFC)



Advanced Nuclear Fuel Cycles

- ☐Sustainability (resources, waste management)
- Public acceptance
- **D**AFC
 - Recycle fissile resources
 - Minor actinides and long-lived fission products utilization/transmutation
 - Waste amount and radio-toxicity reduction



- Renewed interest in P&T
 - Worldwide efforts to assess its potential as a radioactive waste management option
- □P&T complex technology, requiring
 - Advanced reprocessing → in addition to U, Pu and ¹²⁹I, "P" extracts from the liquid high level waste the minor actinides and long-lived fission products, e.g. ⁹⁹Tc, ⁹³Zr, ¹³⁵Cs, ¹⁰⁷Pd, and ⁷⁹Se
 - Fully new transuranics fuel fabrication plants
 - Innovative transmutation reactors



- Present LWRs are not suited for minor actinides and long-lived fission products utilization/transmutation
 - Safety consideration
 - Plant operation
 - Poor utilization/transmutation capability
- Only specially licensed LWRs can cope with MOX-fuel
 - Special reactor designs (e.g. ABB80+, EPR) required for increased Pu loadings (up to 100%)
 - A combination of these reactor types allows Pu inventory stabilization, albeit with increased minor actinides production



- □Long-term waste radio-toxicity can be effectively reduced only if transuranics are fissioned (utilized) → very hard neutron spectra needed
- ■New transmuter reactor concepts
 - Dedicated fast reactors
 - Accelerator Driven Systems (ADS)
 - Fusion/fission hybrid reactors



- Significant Pu and minor actinides utilization rates can be achieved in symbiotic scenarios
 - LWR-MOX and dedicated fast reactors
 - Fast neutron spectrum ADS for minor actinides utilization
 - Very high thermal flux ADS could also provide significant transuranics transmutation yields



- □ Long-lived fission product transmutation difficult:
 - Occur in elemental mixtures (different isotopes of the same element) → isotopic separation required
 - Transmutation yields small because of very low capture cross sections in thermal neutron fields
 - → dedicated reactors required with very high loadings and/or high thermal flux levels



ADS Technology

- □ADS couples spallation source with sub-critical core
- ☐ The basic idea is to make use of the additional flexibility offered by the excess neutrons produced by the spallation source to
 - Produce energy
 - Transmute radio-toxic isotopes
 - Breed fissile material
- ☐Spallation source
 - High energy proton beam on heavy nuclide target producing hard neutrons
 - Less effective than fission neutron source



ADS Technology, cont'd

Spallation neutrons more "expensive", e.g. for Pb target:

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200 MeV → spallation target → 200 \times \eta_{th} \times \eta_e \times \epsilon = 200 \times 0.4 \times 0.5 \times 0.5 = 20 MeV plus 200 \times \eta_{th} \times \eta_e \times Z/E_p = 200 \times 0.4 \times 0.5 \times 25/10^3 = 1 hard neutron (φ* → 1.5 hard neutrons) → compared to fission (producing 200 MeV plus 3 hard neutrons), spallation source needs 180 MeV to produce ½ the number of hard neutrons
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- η_{th} efficiency of thermal to electrical energy transformation η_{e} efficiency of electric energy to proton current transformation ε fraction of incident protons having kinetic energy \ge spallation nucleus dissociation energy
- Z neutrons produced in Pb by each proton of energy E_p



ADS Technology, cont'd

- □Power control is possible up to a certain extent via the proton beam current
- □Should the accelerator have enough reserve, even burnup compensation could be done via the accelerator
- Level of sub-criticality can be chosen, within the technological limits set by the accelerator, larger than $\beta_{eff} \rightarrow$ beneficial to power control and safety
- □Potential advantages → enhanced safety and flexibility
- Potential advantages need substantiation



ADS Technology, cont'd

- ☐ Justification for using "expensive neutron source"
 - Improvement of the dynamics behavior
 - Enhanced flexibility
- □R&D efforts aiming at substantiating the potential of ADS and studying their role in innovative reactor and fuel cycle strategies that include systems for large-scale utilization and transmutation of minor actinides and long-lived fission products



Status of ADS R&D: European Union

- EURATOM 6th (2002 2006) and 7th (2007 2011) Framework Programmes (FPs)
- EUROTRANS project → funded with a total of EUR 45 million, EC contributed 23 million
- Objectives
 - Preliminary design → MYRRHA / XT-ADS (experimental ADS, 50 – 100 MWth)
 - Conceptual design

 European Transmutation Demonstrator (ETD, several hundred MWth, modular)
- Major activities
 - MEGAPIE
 - MYRRHA / XT-ADS project
 - GUINEVERE experimental facility
 - FASTEF and CDT (Central Design Team)



Status of ADS R&D: European Union, cont'd

- ☐ MEGAPIE **MEGA**watt **Pl**lot **E**xperiment
- Joint effort by 6 European Institutes (PSI, FZK, CEA, SCK•CEN, ENEA, CNRS) plus JAEA (Japan), DOE (USA) and KAERI (Rep. of Korea) to demonstrate
 - Design, manufacturing, safe operation, and dismantling of a liquid Pb-Bi eutectic target for high power spallation and ADS applications
 - Assess the target's neutronics performance
 - Collect material data in view of establishing a data base for liquid Pb-Bi eutectic targets
- MEGAPIE was the first liquid Pb-Bi eutectic target operated in the Megawatt regime
 (0.8 MW provided by the PSI proton accelerator)
 - Successfully irradiated from August until December 2006 at the Swiss Spallation Neutron Source (SINQ) at PSI
 - MEGAPIE received a beam charge of 2.8 Ah of 575 MeV protons
- ☐ Dismantling and PIE planned from summer 2009 on



Status of ADS R&D: European Union, cont'd

- ■MYRRHA / XT-ADS demonstrates the feasibility of transmutation with ADS
 - 600 MeV / 2.5 mA or 350 MeV / 5 mA LINAC
 - Pb-Bi eutectic spallation target and coolant
 - Mixed U-Pu oxide fuelled sub-critical core
 - SCK-CEN aligned MYRRHA (full scale ADS demonstrator) R&D activities with XT-ADS efforts:
 - ✓ Investigation of design cliff edges
 - ✓ Demonstration of accelerator components
 - √ Thermal hydraulics design of spallation target
 - ✓ Experimental coupling of a fast sub-critical core with a neutron source in GUINEVERE



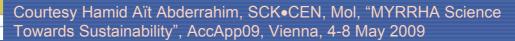
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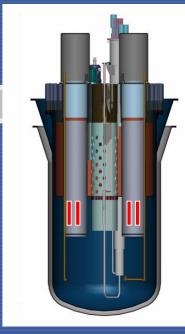
MYRRHA/XT-ADS

1962
BR2
Highest Flux
Material Test
Reactor in Europe



2020
World Premiere
Demonstration
of Nuclear Waste
Transmutation





MYRRHA / XT-ADS: Fast Spectrum Test Facility

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards Sustainability", AccApp09, Vienna, 4-8 May 2009



1962

BR₂

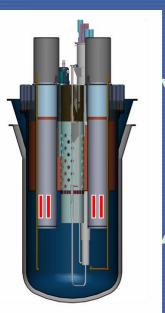
Fast Neutron Material Test Reactor (fission) LWR (GEN II/GEN III) Fuel Testing **Irradiation Services:**

- Medical RI
- Si doping
- Others









2020

MYRRHA / XT-ADS Very Fast
Neutron
Material
Test Reactor
(fission + fusion)

ADS-Demo

P&T Test

Fuel testing for GEN IV

GEN IV Lead Fast Reactor Technology Demonstration

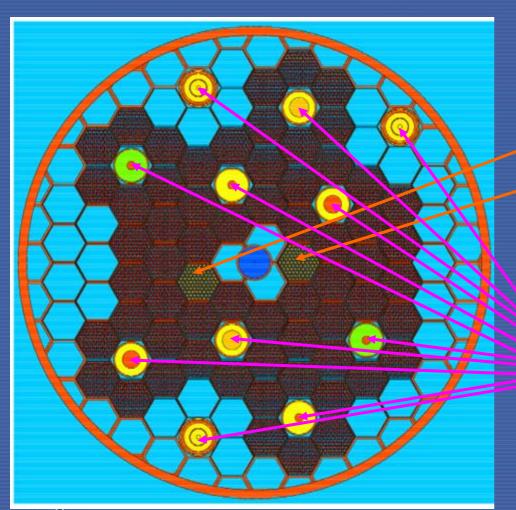
Irradiation Serv. :

- Medical RI
- Si doping
- Others



MYRRHA / XT-ADS: Flexible Experimental Facility

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards Sustainability", AccApp09, Vienna, 4-8 May 2009



Minor Actinides test assemblies

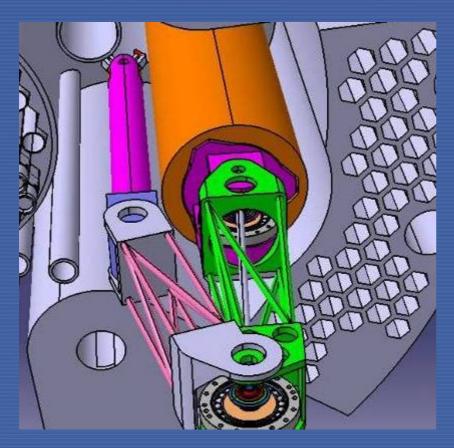
Experimental rigs:

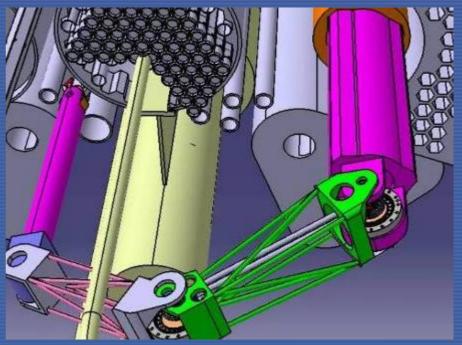
- dedicated contents
- dedicated irradiation



MYRRHA / XT-ADS: In Vessel Remote Handling

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards Sustainability", AccApp09, Vienna, 4-8 May 2009





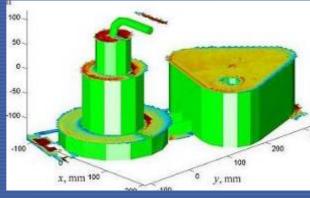


MYRRHA/XT-ADS:

Ultrasonic Imaging in Pb-Bi Eutectic at 300 °C

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards Sustainability", AccApp09, Vienna, 4-8 May 2009

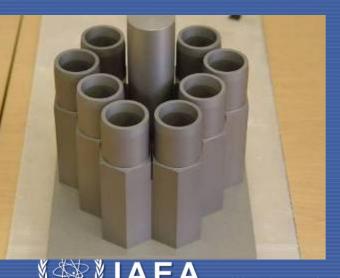


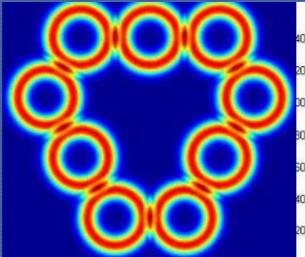


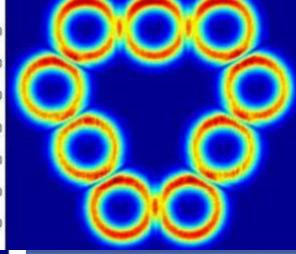
Mockup

Simulated

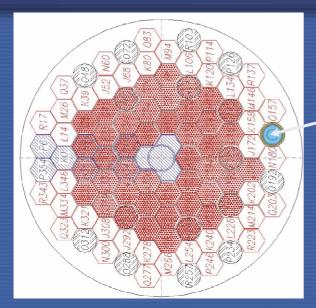
Measured

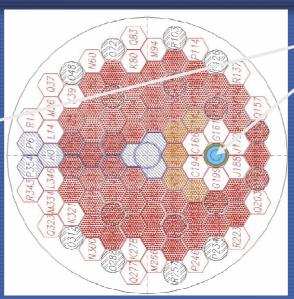






MYRRHA / XT-ADS: Radio-Isotopes Production





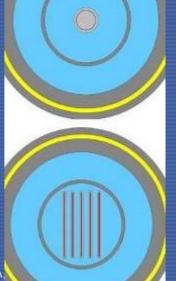
Radio-isotopes production

natir capsule → 192ir

²³⁵U plates → ⁹⁹Mo

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards, Sustainability", AccApp09, Vienna, 4-8 May 2009





MYRRHA / XT-ADS: Project Schedule

Courtesy Hamid Aït Abderrahim, SCK•CEN, Mol, "MYRRHA Science Towards Sustainability", AccApp09, Vienna, 4-8 May 2009

2009-2011

Detailed Engineering Design 2012-2013
Techn.specs
Call for
Tenders
and Awards

2014-2016

Construction of components and Civil Works

2017

Components assembly on site

2018-2019

Commissioning

















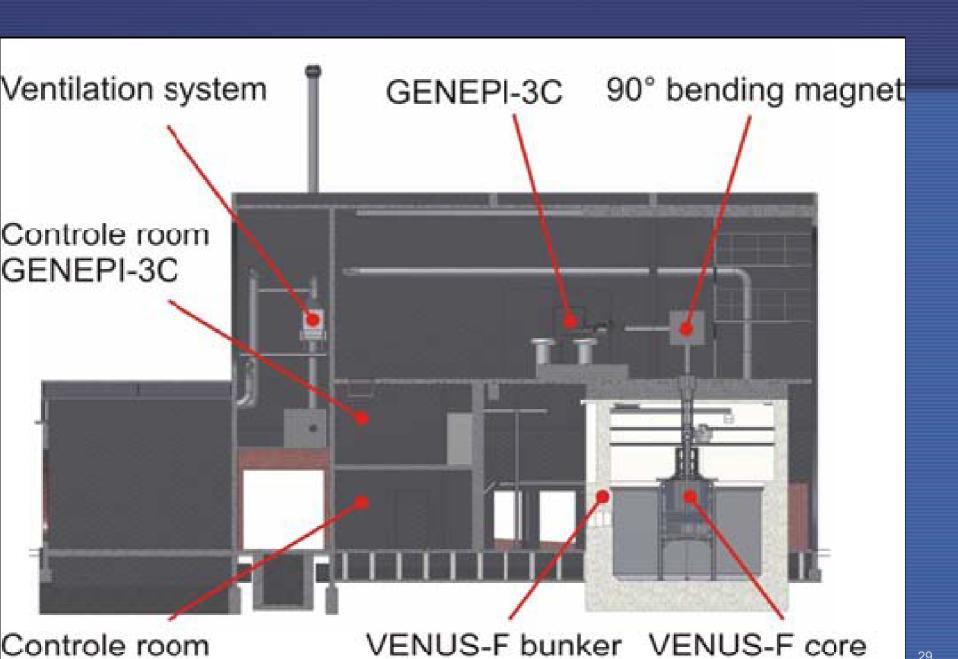


oplications of High Intensity Proton Accelerators, Fermilab, Batavia,

Status of ADS R&D: European Union, cont'd

- ☐ GUINEVERE (Generator of Uninterrupted Intense NEutrons at the lead VEnus REactor)
 - Experimental facility allowing physics experiments and technological research under conditions representative for XT-ADS
 - Deuteron GENEPI-3C accelerator operating in pulsed and continuous mode
 - Ti³H target producing 14.1 MeV neutrons
 - Zero-power fast sub-critical 30% ²³⁵U enriched metallic U fuelled core in Pb matrix





VENUS-F

Courtesy P. Baeten, SCK•CEN, Mol

Status of ADS R&D: European Union, cont'd

- GUINEVERE studies of on-line reactivity monitoring techniques at various sub-criticality levels
 - Current-to-flux reactivity monitoring (GENEPI-3C in continuous mode, representative for power ADS)
 - Time dependent neutron spectra measurements (after beam interruptions) → prompt decay method, source jerk, etc



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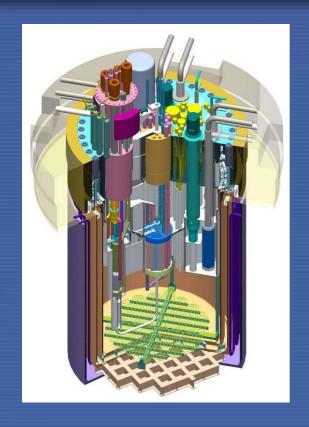
Status of ADS R&D: European Union, cont'd

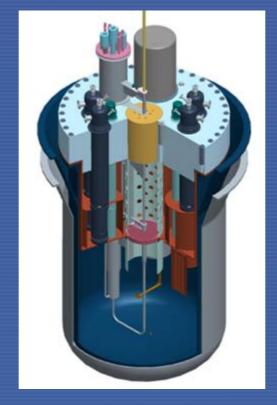
DFASTEF

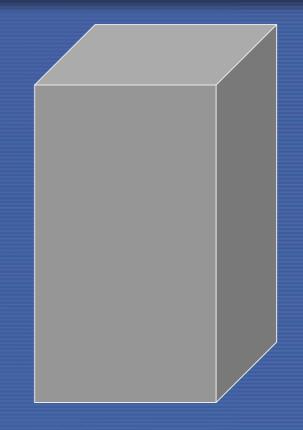
- Builds on MYRRHA / XT-ADS
- Flexible fast spectrum irradiation facility
- A full step ADS demo facility and P&T testing facility
- Contributes to the demonstration of heavy liquid metal technology
- □CDT (Central Design Team) Project
 - Set up of a centralised multi-disciplinary team at SCK•CEN in Mol
 - Produce advanced design of a flexible fast spectrum irradiation facility operating in both sub-critical (ADS) and critical mode
 - Create the nucleus of the "Owner Engineering Team" for the realization of MYRRHA / FASTEF



CDT, MYRRHA/FASTEF, cont'd







MYRRHA 2005

 \Rightarrow

XT-ADS 2009

 \Rightarrow

FASTEF 2012

Courtesy D. De Bruyn, SCK•CEN



Status of ADS R&D: Belarus

- □ Joint Institute for Power and Nuclear Research SOSNY (JIPNR-SOSNY) → YALINA experimental program
- □ Contributes to EUROTRANS and to an ISTC project (HEU to LEU conversion)
- Objectives
 - Minor actinides and long-lived fission product transmutation in fast spectrum sub-critical facilities
 - ADS physics

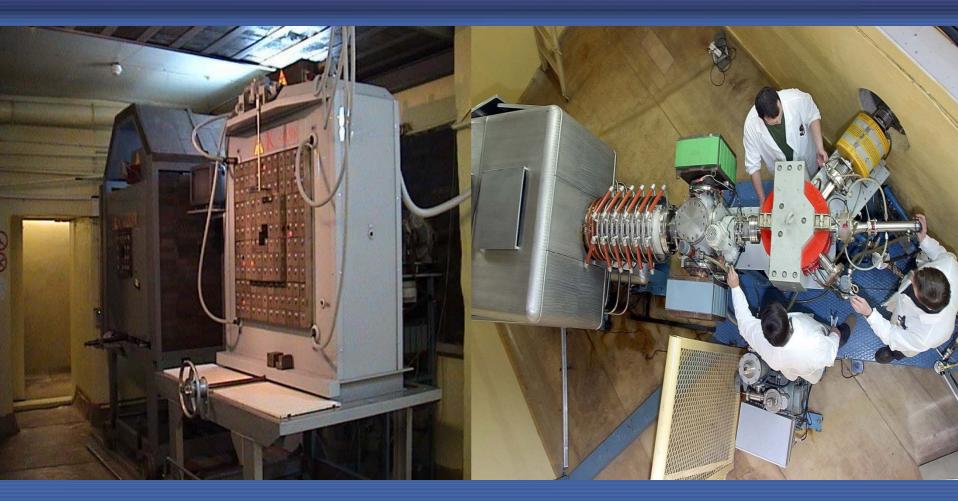


Status of ADS R&D: Belarus (2/2)

- □ Research activities (YALINA-BOOSTER configuration)
 - Development and testing reactivity monitoring techniques used in power ADS
 - Investigation of spatial kinetics of sub-critical systems driven by external neutron sources
 - Measurement of transmutation reaction rates
 - Maintenance and operation characteristics of sub-critical systems driven by external neutron sources



YALINA Facility, JIPNR-SOSNY, Belarus



Courtesy of A. Kiyavitskaya, JIPNR-SOSNY



YALINA Targets

Target diameter [mm]	45	230
Rotation speed [rpm]	560	560
Beam current [mA]	1 – 2	1 - 12
Neutron energy [MeV]	2.5 / 14	2.5 / 14
(D-D / D-T target)		Courtesy of A. Kiyavitskaya, JIF

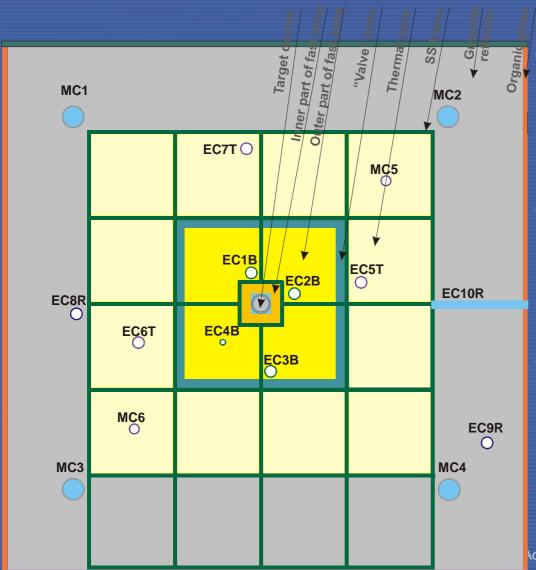
Courtesy of A. Kiyavitskaya, JIPNR-SOSNY







YALINA Booster



EC1B-EC4B: experimental channels in fast zone [metallic U (90%) and UO₂ (36%) in Pb matrix) EC5T-EC7T: experimental channels in thermal zone [UO2 (10%) in polyethylene matrix EC8R-EC10R: experimental channels in reflector

Courtesy of A. Kiyavitskaya, JIPNR-SOSNY

Status of ADS R&D: India

- Objectives of ADS R&D programme
 - P&T as part of advanced fuel cycles
 - Fissile material breeding

 thorium utilization
- Nuclear data
- Code development for high energy particle transport
- High power proton accelerator technology
- □ 14-MeV D-T neutron source coupling with sub-critical reactor (water cooled, natural uranium fuelled)



Status of ADS R&D: India, cont'd

- Spallation target systems and heavy liquid metal thermal hydraulics
 - Pb-Bi eutectic loop with simulated proton beam window heating (plasma torch and electron beam)
 - Corrosion testing
 - Validation and qualification of Computational Fluid Dynamics codes
- ☐Sub-critical core design
 - Thorium fuel utilization in sub-critical systems
 - Experimental reactor offering the flexibility of being transformed into a sub-critical system driven by a spallation source



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Status of ADS R&D: Japan

- □ Japan Atomic Energy Agency (JAEA)
 objective of R&D on ADS → transmutation of long-lived radioactive nuclides
- ☐Sub-critical core design studies: 800 MWth Pb-Bi eutectic cooled ADS
 - Four-zone core concept (limit operating temperatures below 500°C and thus improve the compatibility with Pb-Bi eutectic)
- Corrosion tests for various structural material candidates in low oxygen concentration condition at 450°C and at 550°C



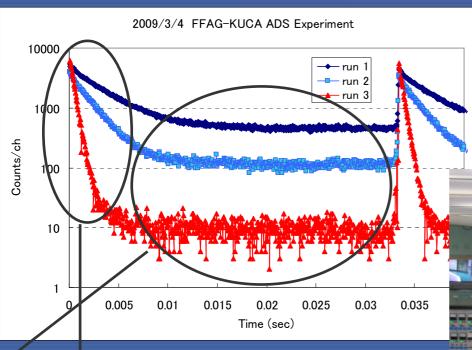
Status of ADS R&D: Japan, cont'd

- □TEF (Transmutation Experimental Facility) design studies → within the framework of J-PARC (Japan Proton Accelerator Research Complex)
 - Handling of minor actinides bearing fuel
 - Remote handling system design



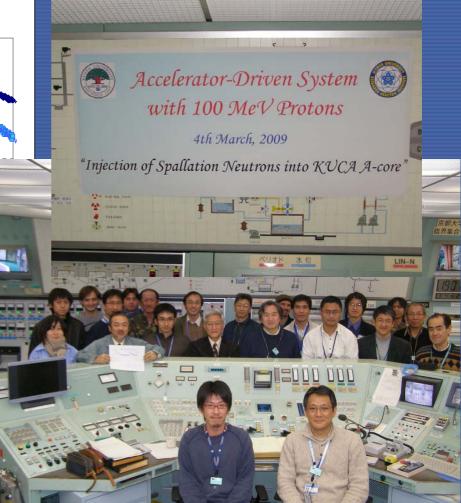
Subcritical Experiments at Kyoto University Critical Assembly (KUCA)

Courtesy of C.H. Pyeon, Kyoto University Research Reactor Institute



Spallation neutrons from target

Delayed neutrons in core





IAEA Activities

- □ Framework → Technical Working Group on Fast Reactors (TWG-FR)
 - Information exchange on fast reactor and transmutation system (e.g. ADS) scientific and technical topics
 - Collaborative R&D (Coordinated Research Projects, CRPs)
 - Membership: Belarus, Brazil, China, France, Germany, India, Italy, Japan, Kazakhstan, Republic of Korea, Russia, Switzerland, UK, and USA; EU (EC), ISTC, and OECD/NEA; observers: Belgium, Sweden



- □ Advanced Workshop on "Model Codes for Spallation Reactions", collaboration with ICTP, 4 8 Feb 2008, Trieste, Italy
 - Intern. Nucl. Data Committee Report 0530
- ■Workshop on "Nuclear Reaction Data for Advanced Reactor Technologies", collaboration with ICTP, 19 – 30 May 2008, Trieste, Italy
 - Lecture notes on http://cdsagenda5.ictp.trieste.it/ full_display.php?smr=0&ida=a07153



- ☐ School on "Physics, Technology and Applications of Innovative Fast Neutron Systems"
 - IAEA's Department of Nuclear Energy and Department of Nuclear Sciences and Applications, in collaboration with ICTP, 9 – 20 November 2009, Trieste, Italy



- □ International Topical Meeting on "Nuclear Research Applications and Utilization of Accelerators", Vienna, 4 8 May 2009
 - Organized by IAEA's Department of Nuclear Energy and Department of Nuclear Sciences and Applications, collaboration with ANS
 - Preliminary proceedings (not for citation yet, username and password: accapp09) on

www-naweb.iaea.org/napc/physics/accapp09/login2.html



- International Conference on "Fast Reactors and Associated Fuel Cycle – Challenges and Opportunities FR09", 7 – 11 Dec 2009, Kyoto
 - Organized by IAEA's Department of Nuclear Energy
 - Hosted by the Japan Atomic Energy Agency
 - Announcement and registration www-pub.iaea.org/MTCD/Meetings/ Announcements.asp?ConfID=35426





- □ Coordinated Research Project on "Studies of Innovative Reactor Technology Options for Effective Incineration of Radioactive Waste" (2003 2008)
 - 17 institutions in 13 Member States & EC (JRC)
 - Transient behaviour of advanced transmutation systems, both critical and sub-critical
 - Papers at PHYSOR 2006, ICENES 2007, and GLOBAL 2007
 - Final CRP report to be published in 2009



- □ Coordinated Research Project on "Analytical and Experimental Benchmark Analyses of Accelerator Driven Systems" (2005 2010)
 - 27 institutions in 18 IAEA Member States
 - Computational and experimental benchmarking >
 work domains: YALINA Booster, Kyoto University
 Critical Assembly (KUCA), TRADE, FEAT, TARC,
 ADS kinetics analytical benchmarks, actinides
 cross sections, spallation targets, ADS
 performance
 - Papers at AccApp2007, and PHYSOR2008



Conclusions

- Renewed interest in nuclear energy
- Sustainability → spent fuel utilization and breeding returning to centre stage → fast reactor necessary linchpin
- Significant reduction of the total Pu and minor actinides inventory → closed fuel cycle strategy → removal of minor actinides from liquid high-level waste before vitrification → achieved by combining advanced MOX fuelled LWRs and dedicated fast reactors
- □ Significant radio-toxicity reduction levels → innovative fuel cycles → fast neutron ADS → vastly enhanced core loadings flexibility, and ability to accommodate very high transuranics and minor actinides inventories) → drastic minor actinides depletion



ADS an option considered for P&T



Conclusions, cont'd

- Achieving the full potential of fast neutron system and closed fuel cycle technologies with regard to both efficient utilization of the fissile resources and waste management requires continued advances in research and technology development to ensure improved economics, and maintain high safety levels with increased design simplification
- IAEA assists Member States considering innovative fast neutron system technology options by providing an umbrella for information exchange and collaborative R&D to pool resources and expertise



For more information, please visit http://www.iaea.org/inisnkm/nkm/aws/fnss/index.html

Thank You!

